



Entergy

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December 19, 1996

2CAN129604

U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Station P1-137
Washington, DC 20555

Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Technical Specification Change Request Concerning
Reactor Coolant System Volume

Gentlemen:

Attached for your review and approval is a proposed amendment to the Arkansas Nuclear One, Unit 2 (ANO-2) Technical Specifications. The proposed amendment deletes the specific value for the total reactor coolant system (RCS) volume in specification 5.4.2. This information is currently included in the ANO-2 Safety Analysis Report (SAR). This change is needed to support a steam generator tube plugging limit above 13%. This change is consistent with NUREG-1432, Revision 1, improved "Standard Technical Specifications for Combustion Engineering Plants" (ISTS).

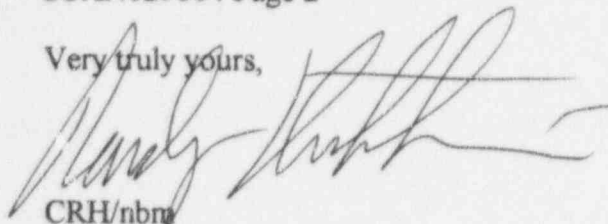
The proposed change has been evaluated in accordance with 10CFR50.91(a)(1) using criteria in 10CFR50.92(c) and it has been determined that this change involves no significant hazards considerations. The bases for these determinations are included in the attached submittal.

This was one of several changes previously requested in correspondence dated August 23, 1996 (2CAN089609). By letter dated October 28, 1996 (2CNA109604), the Staff informed Entergy Operations that the requested review schedule could not be met and requested that the specific changes needed to support the upcoming refueling outage be submitted separately. Entergy Operations requests that the effective date for this change be within 30 days of issuance. Although this request is neither exigent nor emergency, your prompt review is requested prior to the next ANO-2 refueling outage (2R12) which is currently scheduled to begin April 25, 1997.

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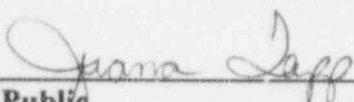
Very truly yours,



CRH/nbr
Attachments

To the best of my knowledge and belief, the statements contained in this submittal are true.

SUBSCRIBED AND SWORN TO before me, a Notary Public in and for Johnson
County and the State of Arkansas, this 19 day of December, 1996,



Notary Public
My Commission Expires 11-8-2000



cc: Mr. Leonard J. Callan
Regional Administrator
U. S. Nuclear Regulatory Commission
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ATTACHMENT

TO

2CAN129604

PROPOSED TECHNICAL SPECIFICATION

AND

RESPECTIVE SAFETY ANALYSES

IN THE MATTER OF AMENDING

LICENSE NO. NPF-6

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT TWO

DOCKET NO. 50-368

DESCRIPTION OF PROPOSED CHANGES

Specification 5.4.2 which specifies the total water and steam volume of the RCS is being deleted from the technical specifications.

BACKGROUND

This change was one of several changes that was previously requested in correspondence dated August 23, 1996 (2CAN089609). In this correspondence, ANO had proposed relocating a major portion of the design features section of the technical specifications to licensing basis documents controlled under the provisions of 10CFR50.59. This proposal was consistent with the format of the ISTS which ANO-2 has committed to convert to in correspondence dated June 21, 1995 (0CAN069502). By letter dated October 28, 1996 (2CNA109604), the Staff informed Entergy Operations that the requested review schedule could not be met and requested that the specific changes needed to support the upcoming 2R12 refueling outage be submitted separately.

DISCUSSION OF CHANGE

The proposed amendment request deletes the total RCS volume from the technical specifications in accordance with the ISTS. The ANO-2 SAR currently contains the RCS volume information. The technical specification information with respect to RCS volume is being removed because it is already controlled in a licensee controlled document under the requirements of 10CFR50.59.

The RCS volume in the design features section of the technical specifications is a nominal hot zero power value with a tolerance for uncertainties. RCS volumes consistent with standard safety analysis methodologies which appropriately account for plugged steam generator tubes are used in the ANO-2 safety analyses. The RCS volume design information is currently given in SAR Tables 5.1-1, 5.1-2, 5.5-2, 5.5-6, and 6.2-8 which is controlled via the 10CFR50.59 process.

The technical specification nominal value for RCS volume contains no allowance for steam generator tube plugging. The technical specification nominal value for RCS volume minus uncertainties corresponds to having approximately 13% average length steam generator tubes plugged. This change is needed to support a steam generator tube plugging limit above approximately 13%. In order to accommodate greater than 13% steam generator tube plugging, a change to RCS volume would be necessary. Future changes to the RCS volume would then be evaluated under 10CFR50.59.

DETERMINATION OF NO SIGNIFICANT HAZARDS CONSIDERATION

An evaluation of the proposed change has been performed in accordance with 10CFR50.91(a)(1) regarding no significant hazards considerations using the standards in 10CFR50.92(c). A discussion of these standards as they relate to this amendment request follows:

Criterion 1 - Does Not Involve a Significant Increase in the Probability or Consequences of an Accident Previously Evaluated.

This proposed change allows the relocation of the reactor coolant system volume in the design features section of technical specifications to the safety analysis report. Future changes will be controlled under 10CFR50.59. This change is considered administrative in nature. Appropriate values of reactor coolant system volume are used in the safety analyses. This change does not affect any system or component functional requirements. The operation of the plant is not affected by this change.

Therefore, this change does not involve a significant increase in the probability or consequences of any accident previously evaluated.

Criterion 2 - Does Not Create the Possibility of a New or Different Kind of Accident from any Previously Evaluated.

The relocation of existing requirements from the technical specifications to another licensee controlled document is administrative in nature. This change does not modify or remove any plant design requirement. The proposed change will not affect any plant system or structure, nor will it affect any system functional or operability requirements. Therefore, no new failure modes are introduced as a result of this change.

Therefore, this change does not create the possibility of a new or different kind of accident from any previously evaluated.

Criterion 3 - Does Not Involve a Significant Reduction in the Margin of Safety.

The proposed amendment request relocates the coolant system volume located in the technical specifications design feature section to another licensee controlled document, the ANO-2 Safety Analysis Report, which is controlled under 10CFR50.59. The proposed change is administrative in nature because the design requirements for the facility remain the same. The proposed change does not represent a change in the configuration or operation of the plant.

Therefore, this change does not involve a significant reduction in the margin of safety.

Therefore, based upon the reasoning presented above and the previous discussion of the amendment request, Entergy Operations has determined that the requested change does not involve a significant hazards consideration.