

# N.R.C. OPERATING DATA REPORT

DOCKET NO. 50-318  
 DATE 2/1/85  
 COMPLETED BY CLIMER  
 TELEPHONE 616-465-5901

## OPERATING STATUS

1. Unit Name D. C. Cook Unit 1  
 2. Reporting Period JUL 85 (notes)  
 3. Licensed Thermal Power (MWt) 3250  
 4. Name Plate Rating (Gross MWe) 1152  
 5. Design Electrical Rating (Net MWe) 1030  
 6. Maximum Dependable Capacity (GROSS MWe) 1056  
 7. Maximum Dependable Capacity (Net MWe) 1020  
 8. If Changes Occur in Capacity Ratings (Items no. 3 through 7) Since Last Report Give Reasons

9. Power Level To Which Restricted, If Any (Net MWe)  
 10. Reasons For Restrictions, If Any:

	This Mo.	Yr. to Date	Cumm.
11. Hours in Reporting Period	744.0	5087.0	92759.0
12. No. of Hrs. Reactor Was Critical	0.0	1868.0	67361.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	463.0
14. Hours Generator on Line	0.0	1856.2	66217.7
15. Unit Reserve Shutdown Hours	0.0	0.0	321.0
16. Gross Therm. Energy Gen. (BWH)	0	5418521	193588025
17. Gross Elect. Energy Gen. (MWH)	0	1761840	63533730
18. Net Elect. Energy Gen. (MWH)	0	1694853	61125948
19. Unit Service Factor	0.0	36.5	73.0
20. Unit Availability Factor	0.0	36.5	73.0
21. Unit Capacity Factor (MDC Net)	0.0	32.7	66.1
22. Unit Capacity Factor (DER Net)	0.0	32.3	63.5
23. Unit Forced Outage Rate	0.0	0.0	7.2
24. Shutdowns Scheduled over Next Six Months (Type, Date, and Duration):			

25. If Shut Down At End of Report Period, Estimated Date of Startup:  
 Shut down for 10 yr. refueling outage. Estimate restart 8/28/85  
 26. Units in Test Status (Prior to Commercial Operation):

Forecast Achieved

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

8510010496 850731  
 PDR ADOCK 05000315  
 R PDR

IE24  
 11

# AVERAGE DAILY POWER LEVEL (MWe-Net)

DOCKET NO. 50-315  
UNIT ONE  
DATE 8/1/85  
COMPLETED BY CLIMER  
TELEPHONE 616-465-5901

MONTH JUL 85

DAY	AVERAGE DAILY POWER LEVEL	DAY	AVERAGE DAILY POWER LEVEL
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1985

DOCKET NO. 50-315  
 UNIT NAME D.C. Cook Unit 1  
 DATE 8-9-85  
 COMPLETED BY B.A. Svensson  
 TELEPHONE 616/465-5901

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
246	850406	S	744	B&C	2	N.A.	ZZ	ZZZZZZ	The Unit was removed from service on 850406 for the scheduled ten-year ISI and Cycle VIII - IX Refueling Outage. The core has been reloaded. The outage has been extended to complete major required design changes. The estimated return to service date is September 1, 1985.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A Equipment Failure (Explain)  
 B Maintenance or Test  
 C Refueling  
 D Regulatory Restriction  
 E Operator Training & License Examination  
 F Administrative  
 G Operational Error (Explain)  
 H Other (Explain)

<sup>3</sup>  
 Method:  
 1 Manual  
 2 Manual Scram.  
 3 Automatic Scram.  
 4 Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I - Same Source

(9/77)

## UNIT SHUTDOWNS AND POWER REDUCTIONS

### INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely<sup>1</sup>. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

**NUMBER.** This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

**DATE.** This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

**TYPE.** Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

**DURATION.** Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

**REASON.** Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

**METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER.** Categorize by number designation

<sup>1</sup>Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

**LICENSEE EVENT REPORT =** Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

**SYSTEM CODE.** The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

**COMPONENT CODE.** Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- If a component failed, use the component directly involved.
- If not a component failure, use the related component; e.g., wrong valve operated through error: list valve as component.
- If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

**CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE.** Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No.: 50-315  
Unit Name: D.C. Cook Unit 1  
Completed By: A. S. Puplis  
Telephone: (616) 465-5901  
Date: August 12, 1985  
Page: 1 of 2

MONTHLY OPERATING ACTIVITIES - JULY, 1985

HIGHLIGHTS:

The Unit is currently in the 10 Year ISI and Cycle 9 Refueling Outage.

There was no electrical generation in the month of July.

SUMMARY:

- 7-05-85 At 0208, the Core Barrel was placed in the reactor vessel.
- 7-11-85 At 0515, the reactor coolant system was at half loop.
- 7-17-85 At 1645, an Unusual Event was declared due to both plant batteries (1AB and 1CD) being declared inoperable. The Unusual Event was terminated at 1320 on 7-20-85.
- 7-22-85 At 1051, Train "B" blackout testing was commenced. Blackout testing was completed at 0040 on 7-27-85.
- 7-23-85 At 0536, Mode 6, Refueling, was entered. Core reload began at 0602.
- At 1645, an Unusual Event was declared, halting fuel movement. The cause of the Unusual Event was an "inoperable" East Essential Service Water Pump due to measured vibration exceeding ISI and indicated low discharge pressure. The discharge pressure gauge was replaced and the vibration levels determined to be acceptable. The Unusual Event was terminated at 0020 on 7-24-85.
- 7-25-85 At 1453, the containment ventilation unit drain containment isolation valves were found to be open. Containment integrity was immediately re-established.
- 7-26-85 At 1735, core reload was completed.
- 7-28-85 At 0602, the reactor vessel closure head was set.

Docket No.: 50-315  
Unit Name: D.C. Cook Unit 1  
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Page: 2 of 2

7-29-85 At 0230, upon confirmation of an RCS boron concentration of 1,967 ppm, which is less than the Technical Specification of 2,000 ppm, the operators immediately borated to greater than 2,000 ppm boron. Investigation into the unexplainable boron dilution is continuing.

At 0923, RCS boron was found to be 1,972 ppm, which is less than the Technical Specification of 2,000 ppm. The operators immediately borated to greater than 2,000 ppm. Investigation into the unexplainable boron dilution is continuing.

The Unit 1 Control Room Cable Vault Halon System was inoperable as of 1400 on 4-05-83. The backup CO<sub>2</sub> system for the Control Room Cable Vault remains operable.

DOCKET NO.	50 - 315
UNIT NAME	D. C. Cook - Unit No. 1
DATE	8-9-85
COMPLETED BY	B. A. Svensson

MAJOR SAFETY-RELATED MAINTENANCE

JULY, 1985

- M-1 A leak was found in the boron injection tank manway flange during ISI hydro.024. Manway was removed, inspected and gasket surfaces cleaned. Torqued to 1500 ft. lbs. per Westinghouse specifications.
- M-2 QRV-302 (3" letdown line flow diversion valve) was discovered to have a body/bonnet leak. A new trim assembly, O-rings and packing were installed along with a new B/B gasket. The valve was tested for operability.



**INDIANA & MICHIGAN ELECTRIC COMPANY**

Donald C. Cook Nuclear Plant  
P.O. Box 458, Bridgman, Michigan 49106

August 9, 1985

Director, Office Of Management Information  
and Program Control  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 1  
Technical Specification 6.9.1.6, the attached Monthly Operating  
Report for the Month of July, 1985 is submitted.

Sincerely,

  
W. G. Smith, Jr.  
Plant Manager

WGS:ab

Attachments

cc: J. E. Dolan  
M. P. Alexich  
R. W. Jurgensen  
NRC Region III  
B. L. Jorgensen  
R. O. Bruggee  
R. C. Callen  
S. J. Mierzwa  
F. S. VanPelt, Jr.  
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D. R. Hahn  
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J. J. Markowsky  
S. R. Khalil  
J. F. Stietzel  
PNSRC File  
INPO Records Center  
ANI Nuclear Engineering Department

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