

MONTHLY OPERATING REPORT - APRIL 1985

At the beginning of the report period, Oyster Creek was operating at 97% thermal power (650 MWe).

Power was reduced to 440 MWe on April 1 to facilitate repairs to a steam leak which had developed on No. 2 Main Flash Tank Manway gasket. Problems were encountered obtaining proper fit-up of the retaining collar used in conjunction with sealant injection. During the next few days, efforts continued to contain leakage and install the collar. On April 4, power was reduced to approximately 50% to facilitate proper collar fit-up.

On April 8, power was further reduced to accommodate manway gasket replacement. The generator was taken off-line at 0410 hours, reactor power reduced to approximately 8% and mode switch placed in "start-up" position. Reactor pressure was reduced and maintained at 500 psig. A gradual increase in reactor power commenced at 1300 hours following completion of gasket replacement. The mode switch was placed in "run" and the generator placed back in-service at 1756 hours producing 200 MWe.

After placing the generator in-service, excessive noise was noted in vicinity of the east side of 'B' condenser. Investigation revealed that the second stage steam reheater tube side relief valve (V-1-133) had lifted. The valve was manually reseated on April 9. Power was increased to 606 MWe on the morning of April 10 and to 627 MWe that evening. Power was reduced to 606 MWe on the morning of April 11 while investigating a concern with thermal power calculations provided by the Power Shape Monitoring System. Following resolution, power was increased to 650 MWe.

On April 12, a leak developed in the Chlorination System. An "Unusual Event" was declared at 6:15 p.m. The leak was isolated and the "Unusual Event" was terminated at 7:16 p.m. A closed chlorine isolation valve leaked, allowing some liquid chlorine to escape to the atmosphere. The valve was in the closed position, but may not have been fully seated. The leak was isolated within six to seven minutes and the Chlorination System returned to service. Power remained at 650 MWe through April 15.

At 6:00 a.m. on April 16, power was reduced to approximately 58% (370 MWe) to repair a seal leak on 'C' condensate pump. Work could not commence because the pump could not be isolated (due to valve leak-through). The pump was returned to service without repair and reactor power was increased to 96% (650 MWe) on April 17. A submersible pump is being used to divert leakage from the condensate pump seal to the hi/low conductivity tank. This will minimize load on the Radwaste Chemical Waste System. The hi/low conductivity tank can be discharged directly to the Radwaste High Purity System. The pump seal leak will be repaired during the next mini-outage.

On April 24, the acoustic monitor for reactor safety valve NR28P annunciated. The sensitivity on adjacent acoustic monitors was adjusted accordingly, and the sensitivity on NR28P was lowered to clear the alarm. Plant parameters are being monitored and do not indicate safety valve leakage. On April 25, while increasing generator load to 650 MWe, acoustic monitor for NR28M annunciated. Plant load was reduced to 633 MWe and the acoustic monitor alarm for NR28M cleared. Background noise in drywell is suspected. Power was increased to 650 MWe on April 26.

On April 27, a leak developed on No. 2 main flash tank at a hole which was inadvertently drilled during a previous sealant injection repair. Power was reduced to approximately 200 MWe to assess repairs and to collect the leakage. Power was increased to 344 MWe the same day. On April 28, power was reduced and the generator removed from service at 7:28 a.m. to accommodate repairs to No.2 flash tank. Repairs were completed and the generator returned to service at 1:20 p.m. the same day. Power was gradually increased over the next several days to 646 MWe on May 1.

Corrective maintenance performed during the report period included:

- o Returned "A" and "B" hydrogen channels for the drywell H₂/O₂ monitoring system to an operable status after surveillance test failures. "A" oxygen channel remains out-of-service.
- o Completed cleaning 1-1 and 1-2 Reactor Building Closed Cooling Water (RBCCW) heat exchangers and returned to service.
- o Completed leak repairs to V-1-21 and V-1-34 (second stage steam reheater supply valves) and 1-2 drain tank water column valve.
- o Completed repairs to Exhaust Fan EF 1-5 dampers.
- o Completed repairs to No. 3 air compressor to correct a high inner stage pressure condition.
- o Completed repairs/cleaning of 1-1 Turbine Building Closed Cooling Water (TBCCW) heat exchanger.

- o Completed repairs to generator amplidyne and returned to service. Commutator machining and brush replacement were required.
- o Adjustments to reactor safety valve acoustic monitors (VMS System) NR28, J, C, G and L were required after discrepancies were identified during surveillance testing.
- o Closure of packing gland leak off valves for V-16-2, 14 and 61 was required to prevent a potential airborne problem in the Reactor Building.
- o Repaired solenoid for air operated valve V-24-37 (1-7 sump isolation valve).
- o Completed replacement of No. 1 demineralized water transfer pump motor and returned to service.
- o Completed reassembly of the AOG System "B" recombiner.
- o Completed repairs to the Chlorination System (service water section) service water booster pump.
- o Adjusted No. 1 main steam bypass valve pilot stem and linkage to allow full closure.
- o Completed overhaul of No's. 2 and 6 intake screens and returned units to service.
- o Completed repair of an oil leak on No. 1 pond pump.
- o Repaired position indication for Reactor Building ventilation valves V-28-1 and 2.

- o A telescoping section of the fuel grapple had to be locked in position because of mechanical binding. This "temporary" repair was necessary to allow movement of fuel bundles within the fuel pool in support of West Valley fuel shipments. Permanent repairs are delayed until grapple can be released for maintenance.

Investigations/repairs in progress at the end of the report period included:

- o Feedwater pump room ventilation system inspections identified a number of deficiencies which are contributing to increased feedwater pump motor temperatures. Repairs to dampers and flow deflectors are in progress. Additional corrective actions are being coordinated by M&C and Plant Engineering.
- o Repairs to No. 3 Turbine Building Closed Cooling Water (TBCCW) pump motor/coupling are in progress.
- o Troubleshooting is in progress to identify the cause of Augmented Off-Gas (AOG) System low flow trips. A high priority has been placed on this item due to technical specification release limits.
- o Repairs to Exhaust Fan EF 1-6 dampers are in progress.
- o The outboard motor bearing failed on "A" control rod drive (CRD) pump. Repairs are scheduled to be completed by May 3.
- o No. 1 dilution pump motor reassembly is in progress following replacement of motor bearing. Replacement of bearing oil seals has delayed returning the unit to service.
- o Repairs to the High/Low conductivity diverter valve are required to continue Air-Bump-Rinse-Operations (ABRO's) of the condensate demineralizers. ABRO's of condensate demineralizers No. 1, 2 and 6 have been completed.
- o Repairs to plant recorders continued as required. Plant Engineering reported erratic printing problems were due to dirty and worn stepping switches. Switches in a number of recorders were replaced and recorder performance is being monitored. Chart drive problems continue to occur periodically.

- o Inspections of ventilation system air washers are in progress. Maintenance requests (Short Forms) are being developed to resolve problems identified.
- o During the report period, a step change in feedwater flow occurred. Plant Engineering and Technical Functions have concluded the flow change was due to a small feedwater heater tube leak. Further investigation is required to determine which feedwater heater is affected.
- o Extensive inspections performed by Plant Engineering identified numerous condenser vacuum leaks. Significant leaks were identified on Turbine cross-around Relief Valve expansion joints. Repair activities are in progress.
- o Torus water level has been slowly increasing (approximately 0.5 inches in the last 25 days). Operations personnel are attempting to determine the source of in-leakage.

NOTE: Gross generator output for the month of April was 386,820 MWH's.

NOTE: At the end of the report period the drywell unidentified leak rate was 1.05 gpm and the identified leak rate was 4.15 gpm.

The following Licensee Event Reports were submitted during the month of April 1985:

Licensee Event Report 50-219/85-007 - On March 20, 1985, during a routine Technical Specification surveillance, it was discovered by the Plant Chemistry Department that the outside Floor Drain Sample Tank was being used but had not been sampled since March 13, 1985. This is in violation of Technical Specifications which require this tank to be sampled every 72 hours unless it has been valved out of service after determining its radioactive content. Upon discovering that the tank was being used but not sampled, a sample was taken to confirm that the tank did not exceed the applicable Technical Specification maximum curie limit.

Licensee Event Report 50-219/85-008 - A Plant Engineering review of Technical Specification Amendment 80 found that existing procedures did not meet the new Technical Specification requirements and calibration tolerances. Existing calibration documentation was reviewed for degraded voltage relays and the degraded voltage relay timers. Although they were found to be within the acceptable tolerances stated in the existing procedures, the procedures had not been revised to incorporate the recently issued Technical Specification requirements. The amendment was effective on the date of issuance and did not provide for an implementation period in which to revise the procedures. Immediate action was taken to temporarily change the procedures required to ensure compliance with the Technical Specification Amendment.

OPERATING DATA REPORT
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: April 1985
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): N/A
11. REASON FOR RESTRICTION, IF ANY: NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	720.0	2880.0	134593.0
13. HOURS RX CRITICAL	720.0	2221.0	87848.9
14. RX RESERVE SHTDWN HRS	0.0	0.0	469.7
15. HRS GENERATOR ON-LINE	700.4	2099.7	85636.4
16. UT RESERVE SHTDWN HRS	19.6	19.6	22.3
17. GROSS THERM ENER (MWH)	1144000	3524940	140787269
18. GROSS ELEC ENER (MWH)	386820	1213960	47596955
19. NET ELEC ENER (MWH)	371550	1163595	45726160
20. UT SERVICE FACTOR	97.3	72.9	63.6
21. UT AVAIL FACTOR	100.0	73.6	63.6
22. UT CAP FACTOR (MDC NET)	83.2	65.2	54.8
23. UT CAP FACTOR (DER NET)	79.4	62.2	52.3
24. UT FORCED OUTAGE RATE	0.0	25.8	10.2
25. FORCED OUTAGE HRS	0.0	729.5	9680.6
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):	N/A		
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:	N/A		

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET #50-219
UNIT.Oyster Creek #1
REPORT DATEMAY 08, 1985
COMPILED BYWILLIAM J. EMRICH, JR.
TELEPHONE #609-971-4637

MONTH APRIL, 1985

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	522	16.	458
2.	483	17.	601
3.	568	18.	616
4.	439	19.	609
5.	343	20.	612
6.	361	21.	611
7.	365	22.	607
8.	67	23.	605
9.	227	24.	610
10.	578	25.	612
11.	604	26.	617
12.	621	27.	425
13.	626	28.	268
14.	627	29.	562
15.	627	30.	611

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April 1985

DOCKET NO. 50-219
 UNIT NAME Oyster Creek
 DATE 5-3-85
 COMPLETED BY R. Baran
 TELEPHONE _____

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
36	2-2-85	F	785.6	A	Z	NA	ZZ	ZZZZZZ	<p>NOTE 1: The generator was taken off the line for 13.8 hours to replace the No. 2 flash tank manway gasket which failed April 8, 1985.</p> <p>NOTE 2: The generator was taken off the line for 5.9 hours to repair a hole in No. 2 flash tank April 28, 1985.</p> <p>NOTE 3: A 30% reduction in power was required for 1.5 hours due to the temporary unavailability of two circulating water pumps.</p>

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

4
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

5
 Exhibit I - Same Source

REFUELING INFORMATION - April, 1985

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: November 30, 1985

Scheduled date for restart following refueling: June 1, 1986

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes

Scheduled date(s) for submitting proposed licensing action and supporting information:

June, 1985

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core = 560
(b) in the spent fuel storage pool = 1120

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present licensed capacity: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Reracking of the fuel pool is in progress. Four out of ten (10) racks have been installed to date. When reracking is completed, discharge capacity to the spent fuel pool will be available until 1990 refueling outage.