

July 10, 1985

For: The Commissioners

From: T. A. Rehm, Assistant for Operations, Office of the EDO

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING JULY 5, 1985

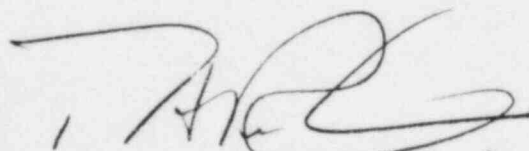
A summary of key events is included as a convenience to those Commissioners who may prefer a condensed version of this report.

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*No input this week.



T. A. Rehm, Assistant for Operations
Office of the Executive Director
for Operations

Contact:
T. A. Rehm, EDO
492-7781

8507180644 850710
PDR COMMS NRCC
WEEKLYINFOREPT PDR

HIGHLIGHTS OF WEEKLY INFORMATION REPORT

WEEK ENDING JULY 5, 1985

Grand Gulf Unit 1

Mississippi Power and Light Company declared Grand Gulf Unit 1 to be in commercial operation on July 1, 1985. The unit is operating at approximately 100% power. The unit has a design electrical rating of 1250 MWe.

Bellefonte Units 1 and 2

TVA announced cutbacks in site staff levels at the Bellefonte nuclear facility from approximately 3300 to 1300 personnel by October 1, 1985. Unit 1 fuel load date is sometime in 1993 and Unit 2 in 1995. Forecasts show that Bellefonte is not needed until this time period.

Nuclear Plant Analyzer (NPA) Calculations for the Davis-Besse Incident

The NPA is being used by NRR to perform "what if" studies for the recent Davis-Besse loss-of-feedwater incident. RES and Idaho National Engineering Laboratory (INEL) support for this special effort has been arranged.

LOW-LEVEL WASTE COMPACTS

The Appalachian Compact bill has cleared the Pennsylvania Senate and been sent to the House.

Although it had appeared that New York State executive and legislative officials were near agreement on a low-level waste management bill, it was not enacted before the summer legislative recess. The New York Legislature reconvenes in October.

OFFICE OF ADMINISTRATION

Week Ending July 5, 1985

ADMINISTRATION OF THE FREEDOM OF INFORMATION ACT

STATUS OF REQUESTS

	<u>Initial Request</u>	<u>Appeal of Initial Decision</u>
Carryovers, 1984	179	23
Received, 1985	472	28
Granted	361	22
Denied	90	10
Pending	200	19

ACTIONS THIS WEEK

Received

Scott A. Klion,
General Electric
Company
(85-461) Request for copies of all records of or relating to communications and/or correspondence from July 26, 1984, to the present between Mozart Ratner and any NRC employee.

Michael K. O'Mealia,
Doc-Search
Associates
(85-462) Request for a copy of a document, "XN-NF-487, 'Results and Analysis of Transient Rod Bundle Boiling Transition Tests,' December 1978," referenced on page 30 of Report No. XN-NF-524(NP).

(An Individual
Requesting
Information)
(85-463) Request for records in her security files.

Susan L. Hiatt,
OCRE
Representative
(85-464) Request for all records resulting from or concerning the meeting between the NRC staff and the Hydrogen Control Owners Group held on or about May 22, 1985.

Jim Pedro,
NUS Corporation
(85-465) Request for copies of summaries for five specified IDCOR/NRC Technical Exchange meetings.

Joseph J. Holonich,
International
Energy Associates
Limited
(85-466) Request for records indicating the inspection manhours expended on an annual basis for Beach Bottom Units 2 and 3 and the Limerick Plants from 1970 through 1980.

CONTACT: J. M. Felton
492-7211

JUL 5 1985

ENCLOSURE A

Received Cont'd

Anthony C. Dicola (85-467)	Request for all surveys conducted by NRC of the building previously owned by Metcoa Company of Pulaski, Pennsylvania, under License STB 1254, and all correspondence between Dollar Savings Bank and NRC regarding that property.
Nina Bell, Nuclear Information and Resource Service (85-468)	Request for all records related to the cost of Industry-Sponsored Probabilistic Risk Assessments.
Steven B. Litzell, Software AG of North America, Inc. (85-469)	Request for four categories of records related to the contract awarded pursuant to Request for Proposal RS-ORM-85-312.
Frank S. Hamblett, Newman & Holtzinger, P.C. (85-470)	Request for a copy of SECY-85-222 entitled "Final Backfit Rule."
Walter L. Robinson, Walter L. Robinson & Associates (85-471)	Request for a copy of the "NRC Policy and Guidance Directive for Lixiscopes and Dual Photon Absorptiometry Equipment Licensing."
James N. Gillman, Blue Cross and Blue Shield Association (85-472)	Request for a list containing the names of NRC employees and their position titles, grade levels, and work locations.

Granted

Ellyn R. Weiss, Harmon, Weiss, & Jordan (84-147)	In response to a request for all records sent by Tim Martin of NRC Staff to the Commissioners related to leak rate calculations at Three Mile Island, informed the requester that records subject to her request are located in the PDR.
Billie P. Garde, Government Accountability Project (84-778)	In response to a request for copies of records regarding inspections that Region IV has conducted into allegations by current or former employees of the Comanche Peak project about the stainless steel liner plates of the spent fuel pool, transfer canal, refueling cavity or any other facility at the site, made available 24 records. Informed the requester that one additional record subject to her request is already located in the PDR.

ENCLOSURE A

JUL 5 1985

Granted Cont'd

Billie P. Garde, Government Accountability Project (85-154)	In response to a request for records generated in connection with all information provided in any form to the Department of Energy Nuclear Power Task Force concerning the Comanche Peak plant, informed the requester that the NRC has no records subject to her request.
James R. Polson (85-208)	In response to a request for 17 specified inspection reports and other records, made available 5 records. Informed the requester that 12 additional records subject to his request are already available at the PDR.
Billie P. Garde, Government Accountability Project (85-212)	In response to a request for records generated in connection with all communications between any persons in the Executive Branch and any NRC Commissioner or staff person working for a Commissioner or any staff member regarding Comanche Peak since June 1, 1984, informed the requester that the NRC has no records subject to this request.
Thomas Sheehan, Jr., Gilbert/ Commonwealth, Inc. (85-224)	In response to a request for a copy of the successful proposal and contract awarded in response to Request for Proposal No. RS-ORM-84-389 entitled "Requirements Analysis, Design, Development and Implementation of an Integrated Management System," made available one record. Informed the requester that the contract has already been made available at the PDR.
Connie M. Lodge (85-285)	In response to a request for data and studies on the changes in disease rates and other health-related problems of the local residents for a period of years before the 1979 TMI accident and the six years since then, made available 28 records. Informed the requester that two additional records subject to her request are already located in the PDR.
William J. Herrmann and David G. Gilbert, Governor's Office of Consumer Services (85-301)	In response to a request for copies of records related to the drafting of IE Bulletins 79-14 and 80-11, made available 60 records. Informed the requesters that four additional documents subject to their request are already located in the PDR.
Norman Aamodt (85-395)	In response to a request for records regarding NRC staff knowledge that several control rods did not fall into the reactor core in the TMI-2 accident in March 1979, made available one record. Informed the requester that one additional record subject to his request is already located in the PDR.

ENCLOSURE A

Granted Cont'd

Joseph M. Corrigan
(85-401)

In response to a request for all records relating to NRC requests for non-military or military personnel, including the Green Beret Forces, to penetrate and or evaluate the physical protection plans of nuclear power facilities within the United States, particularly Units 2 and 3 of the Indian Point plant in Buchanan, New York, made available seven records. Informed the requester that one additional record related to his request is already located in the PDR.

Philip P. Steptoe,
Isham, Lincoln &
Beale
(85-414)

In response to a request for copies of any records provided to Messrs. Herrmann and Gilbert of the Illinois Governor's Office of Consumers Service pursuant to their FOIA request 85-31, informed the requester that the records are available in the PDR.

Barry Scanlon, M.D.,
Psychiatric

In response to a request for records related to ownership or change of ownership of Psychiatric Diagnostics Laboratories of America, Inc., and National Psychopharmacology Laboratory, Inc., of Knoxville, Tennessee, made available one document. Informed the requester that five additional records subject to his request are already located in the PDR.

Neal R. Gross,
Neal R. Gross and
Co., Inc.
(85-436)

In response to a request for copies of the contract and bid abstract for the NRC's most recent court reporting and transcription services contract (NRC-17-85-394), informed the requester that the contract is already in the PDR and that no bid abstract was prepared.

Denied

Lyle Graber,
NUS Corporation
(84-926)

In response to a request that NRC place in the PDR all NRC internal memoranda to Division of Licensing on Beaver Valley Unit 2 from May through November 1984, which provided draft SER, SER sections, and NRC questions or requests for additional information for transmittal to Division of Licensing, made available 75 records. Denied portions of one record, disclosure of which would reveal unclassified safeguards information.

Lynn Cunningham,
The Times-Picayune
Publishing
Corporation
(85-191)

In response to a request for copies of records regarding the March 15, 1985, closed Commission meeting on the Waterford 3 plant, made available three records. Informed the requester that two additional records are already publicly available in the NRC's Public Document Room. Denied two records in their entirety, disclosure of which would tend to inhibit the open and frank exchange of ideas essential to the deliberative process.

ENCLOSURE A

Denied Cont'd

Robert R. Belair,
Kirkpatrick &
Lockhart
(85-199)

In response to a request for copies of records related to severe accident source term data, made available 98 records. Informed the requester that three additional records are already publicly available in the NRC's Public Document Room. Denied 13 records in their entirety, disclosure of which would tend to inhibit the open and frank exchange of ideas essential the the deliberative process.

Harry A. Hammitt,
FOI Services, Inc.
(85-287)

In response to a request for copies of the application, license, and amendments for Syncor Corporation, made available 23 records. Denied portions of five records, disclosure of which would constitute a clearly unwarranted invasion of personal privacy.

Thomas H. Hartman,
Interfacts, Inc.
(85-333)

In response to a request for records containing the names, or other identifiers, of all bidders to solicitation RS-NRR-85-054, the addresses of all bidders, and the name and title of all persons authorized to sign this offer, denied two records in their entirety due to the fact that the contract has not as yet been awarded and the information contained in the two records is predecisional in nature.

ENCLOSURE A

JUL 5 1985

WEEKLY INFORMATION REPORT
DIVISION OF CONTRACTS
WEEK ENDING JULY 5, 1985

PROPOSALS UNDER EVALUATION

RFP No.: RS-RES-85-106

Title: "Piedmont Seismic Reflection Study"

Description: Select a specific seismological problem related to the Southern Piedmont, design a program of seismic reflections and other geographical investigations, execute the program & interpret the resulting data. The information gathered is to be used to advance the state of knowledge on possible causes of seismicity in the Piedmont of Virginia & the Carolinas, particularly with respect to structure at hypocentral depths.

Period of Performance: 2 years with additional one year option.

Sponsor: Office of Nuclear Regulatory Research

Status: The competitive range has been established and negotiations are scheduled for the week of July 8, 1985.

RFP No.: RS-RES-84-114

Title: "Flow of Groundwater and Transport of contaminants through Saturated Fractured Geologic Media from High Level Radioactive Waste (HLW)"

Description: Provide NRC with research products which may support and/or confirm the basis for technical positions on the above subject and to provide a capability sufficient to evaluate DOE documents related to groundwater flow and transport in saturated fractured rocks in which HLW might be stored.

Period of Performance: 3 years

Sponsor: Office of Nuclear Regulatory Research

Status: Best and Final offers received on July 2, 1985 and forwarded to Source Evaluators for review on July 2, 1985.

RFP No.: RS-NMS-85-007

Title: "Facility Systems Analysis Report"

Description: Provide technical support in the form of reports, analytical, and engineering expertise to fully implement the regulatory effectiveness review program for all power reactors.

Period of Performance: 24 months

Sponsor: Office of Nuclear Material Safety and Safeguards

Status: Best and Final offers due on July 17, 1985.

ADMINISTRATIVE MATTER

Two protests were received on June 28, 1985, under IFB No. RS-ADM-85-252 for "Micrographic Services of Source Drawing Documents." The first protest was filed with SBA and the second protest was filed with GAO. This solicitation was totally set aside for small business participation. Both protestors allege that the apparent low bidder is ineligible for award as he does not qualify as a small business under the applicable size standard.

ENCLOSURE A

JUL 5 1985

OFFICE OF NUCLEAR REACTOR REGULATION

ITEMS OF INTEREST

Week Ending July 5, 1985

Grand Gulf Unit 1

Mississippi Power and Light Company declared Grand Gulf Unit 1 to be in commercial operation on July 1, 1985. The unit is operating at approximately 100% power. The unit has a design electrical rating of 1250 MWe.

Catawba Unit 1

On June 29, 1985, Catawba Unit 1 entered commercial operation after completing 100-hour run at 100% power.

Bellefonte Units 1 and 2

TVA announced cutbacks in site staff levels at the Bellefonte nuclear facility from approximately 3300 to 1300 personnel by October 1, 1985. Unit 1 fuel load date is sometime in 1993 and Unit 2 in 1995. Forecasts show that Bellefonte is not needed until this time period.

Waterford 3

At 6:45 p.m. on July 1, 1985, Waterford 3 achieved 100% power for the first time. Transient testing of the plant (100% generator trip and 100% turbine trip-reactor trip) is scheduled for July 8, 1985, with completion of the 100-hour warranty run scheduled for July 14, 1985.

NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT

WEEK ENDING JULY 5, 1985

1. PLANT STATUS

- The facility remains in long term cold shutdown with the Reactor Coolant System (RCS) vented to the reactor building atmosphere and the reactor vessel head and plenum assembly removed.
- The reactor vessel plenum has been removed from the reactor vessel and placed on its storage stand in the deep end of the fuel transfer canal. A dam has been installed between the deep and shallow ends of the fuel transfer canal. The deep end is filled with water to a depth of about 20 feet (about 5 feet above the top of the plenum).
- The modified internals indexing fixture is installed on the reactor vessel flange and is flooded to elevation 327 feet 6 inches (15½ feet above the top of the core region).
- Calculated reactor decay heat is less than 12 kilowatts.
- RCS cooling is by natural heat loss to the reactor building ambient atmosphere. Incore thermocouple readings range from 73°F to 94°F with an average of 83°F. Average cold leg temperature is 58°F.
- The average reactor building temperature is 63°F. The reactor building airborne activity is 7.4 E-9 uCi/cc tritium, and 2.7 E-10 uCi/cc particulate, predominantly cesium 137.

2. WASTE MANAGEMENT

- Both the Submerged Demineralizer System (SDS) and EPICOR II were shutdown during this period.
- Total volume processed through SDS to date is 2,898,836 gallons, and the total volume processed through EPICOR II is 2,480,774 gallons.

3. DOSE REDUCTION/DECONTAMINATION ACTIVITIES

- Shielding of the reactor building air coolers is in progress.
- Average general area radiation dose rate is 36 mrem per hour on the 347' level of the reactor building and is 160 mrem per hour on the 305' level of the reactor building.

ENCLOSURE B

JUL 5 1985

4. ENVIRONMENTAL MONITORING

- EPA sample analysis results show TMI site liquid effluents to be in accordance with regulatory limits, NRC requirements, and the City of Lancaster Agreement.
- TMI water samples taken by the US Environmental Protection Agency at the plant discharge to the river consisted of seven daily composite samples taken from June 15, to June 22, 1985. Gamma scans detected no reactor related radioactivity.
- The Lancaster water sample taken at the water works intake and analyzed by the US Environmental Protection Agency consisted of a seven day composite sample taken from June 16, to June 22, 1985. A gamma scan detected no reactor related radioactivity.
- The NRC outdoor airborne particulate sampler at the TMI Site collected a sample between June 27, and July 3, 1985. No reactor related radioactivity was detected. Analysis showed I-131 and Cs-137 concentrations to be less than the lower limits of detectability.

5. AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES

- Installation of the DWCS continued. Partial DWCS turnover for processing RCS during early defueling is scheduled to be completed in late August.

6. NRC EVALUATIONS IN PROGRESS

- Defueling Water Cleanup System Technical Evaluation (including Revision 6)
- Technical Specification Change Requests numbers 46, 48, and 50
- Equipment Hatch Removal Safety Evaluation
- Recovery Operations Plan Change numbers 27, 29, and 32
- Fuel Canister Technical Evaluation
- Fuel Handling Senior Reactor Operator Training Program
- Defueling Safety Evaluation

7. PROJECTED SCHEDULE OF FUTURE EVENTS

- Start of Defueling: September 1985

8. PUBLIC MEETING

- The next meeting of the Advisory Panel is scheduled for 7:00 PM, July 18, 1985, at the Public Service Building, 201 North Duke Street, Lancaster, PA.

OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS

Items of Interest

Week Ending July 5, 1985

Contractor Report

During the week of July 19, 1985, Mr. William Bland, a contractor working for the Division of Waste Management (WM), will accompany Dr. Michael Bell, Deputy Director, Waste Management on a trip to the DOE Repository Project Offices in: Columbus, Ohio; Hanford, Washington; and Las Vegas, Nevada. Mr. Bland will be discussing his recent report (which will be published as NUREG/CR 4271) "Recommended Safety Reliability, Quality Assurance, and Management Aerospace Techniques with Possible Application by the DOE to the High Level Radioactive Waste Repository Program". Mr. Bland previously briefed the DOE Headquarters staff on the report.

JUL 5 1985

ENCLOSURE C

OFFICE OF INSPECTION AND ENFORCEMENT
Items of Interest
Week Ending July 5, 1985

1. The following Significant Enforcement Actions were taken during the past week:
 - a. EN 85-49, Order Modifying License and a Notice of Violation and Proposed Imposition of Civil Penalty in the Amount of \$6,250 was issued June 28, 1985 to Advanced Medical Systems Inc. (Geneva, OH). This action was based on significant weaknesses in management control of the radiation protection program.
 - b. EN 85-50, a Notice of Violation and Proposed Imposition of Civil Penalty in the amount of \$4,000 was issued July 3, 1985 to Princeton University (Princeton, NJ). This action is based on several violations leading to an individual receiving a skin exposure of 38 rems.
2. The following IE Preliminary Notifications were issued during the past week:
 - a. PNO-I-85-44, Public Service Electric and Gas Company (Salem, Unit 2), Unscheduled Outage.
 - b. PNO-I-85-45, Long Island Lighting Company (Shoreham), Planned Demonstration at Shoreham Site.
 - c. PNO-III-85-54, Toledo Edison Company (Davis-Besse), Loaded Weapon Found During Entry Search.
 - d. PNO-III-85-54A, Toledo Edison Company (Davis-Besse), Loaded Weapon Found During Entry Search (Update).
 - e. PNO-IV-85-28, Centennial Engineering, Inc. (Casper, WY), Loss of Radioactive Material.
 - f. PNO-IV-85-28A, Centennial Engineering, Inc. (Casper, WY), Loss of Radioactive Material. (Update)
 - g. PNO-V-85-37A, Sacramento Municipal Utility District (Rancho Seco) Unisolable Reactor Coolant Leak. (Update)

3. The following IE Information Notice was issued during the past week:

- a. IE Information Notice 85-49, Relay Calibration Problem, was issued July 1, 1985 to all nuclear power reactor facilities holding an operating license or a construction permit.

4. Other Items

a. Quality Assurance

- (1) On July 1 and 2, 1985, representatives of the Quality Assurance Branch, Division of Quality Assurance, Vendor, and Technical Training Programs, met in Boston, MA, with Stone & Webster Engineering Company to inspect checklists associated with the independent design review of the readiness review for Vogtle 1.
- (2) On July 1 and 2, 1985, representatives of the Quality Assurance Branch met in Seattle, WA with Washington Public Power Supply System to discuss the potential application of a readiness review program for WNP-1 and 3.

b. Program Development

In the area of program development, three inspection procedures (IP) dealing with containment integrated leak rate testing were sent to the regions for comment. These IPs, part of the 2515 Operating Reactor Program, were revised to provide improved inspection guidance, to reflect the most recent requirements of 10 CFR 50, Appendix J, and to revise the inspection frequency to reflect the increasing importance of local leak rate testing programs and verifying containment integrity.

c. Incident Response/Emergency Preparedness

IE staff members attended the FEMA NRC Steering Committee meeting on July 2, 1985.

d. Enforcement Conference

Representatives of Enforcement Staff will be in Region II this week to attend enforcement conferences on Oconee, Catawba, and McGuire.

e. Pipe Supports

Representative of Reactor Construction Branch, Division of Inspection Programs, was at Rancho Seco this week to assist Region V with the evaluation of pipe supports related to recent pipe failures.

OFFICE OF NUCLEAR REGULATORY RESEARCH

Items of Interest

Week Ending July 5, 1985

2D/3D Coordination Meeting in Germany, July 18-21, 1985

The Upper Plenum Test Facility (UPTF) is now "buttoned up" and undergoing commissioning tests. Main testing is scheduled to start in December 1985, after a 2-month delay caused by adding a condensate drain system to minimize condensation shocks.

Since the 2D/3D data has potential for use for U.S. designs in assessing models for the proposed Emergency Core Cooling System (ECC) rule change, agreement was reached on procedures to disseminate Quick-Look Reports. Japan Atomic Energy Research Institute (JAERI) will seek approval for these procedures from their funding agency, Science and Technology Agency (STA).

The participants recognized the desirability to perform the official 2D/3D calculations with a frozen version of the TRAC code, as long as the comparison of experimental and calculated data do not show major physical deficiencies. The situation will be reexamined in April 1986, at the next Steering Committee Meeting.

It was agreed to investigate the possibility of performing upper plenum injection (UPI) tests in UPTF. Evaluations will be exchanged by September 1985.

Previous TRAC calculations predicted the existence of radial cross-flow in the Japanese Slab Core Test Facility (SCTF). This cross-flow helps to provide cooling water to the hotter bundles. Differential Pressure measurements in SCTF suggested that significant cross-flow exists, but a more direct measurement was called for by the participants to confirm this phenomena. Thus, JAERI recently installed five special drag disks in SCTF and reported at this meeting the confirmation of significant radial cross-flow, as originally predicted by TRAC.

Combined hot-leg and cold-leg injection tests conducted in Cylindrical Core Test Facility-II (CCTF-II) showed that liquid penetrates down into only a part of the lower power zone in the core, while in the other parts of the core steam flows up. The area of liquid dumping (from the upper plenum into the core) seems to be stochastic in nature. In the regions where steam goes up, bottom quenching is dominant, while in the regions of liquid dumping both bottom and top quenching occurred, leading to about a 50 percent reduction in quench time in these regions. The net flow at the bottom of the core was generally downward for these special tests, although there were considerable fluctuations between small upward flows and large downward flows.

Research Information Letter (RIL) on CCTF-I Results

A RIL on the JAERI CCTF-I test and analysis results has been prepared for transmittal to NRR. These Large-Break LOCA Integral System results for Pressurized Water Reactor (PWR) show that even though steam binding does occur, limiting the liquid inflow into the core to about 1 inch/second, effective core cooling still results from the two-phase mixture flowing through the core. A margin of conservatism of over 500°F in peak clad temperature was confirmed both in the tests and in the TRAC analyses.

Nuclear Plant Analyzer (NPA) Calculations for the Davis-Besse Incident

The NPA is being used by NRR to perform "what if" studies for the recent Davis-Besse loss-of-feedwater incident. RES and Idaho National Engineering Laboratory (INEL) support for this special effort has been arranged.

Complete use of the INEL computer by NRR during specified daily hours for the next few weeks has been offered to expedite the interactive analysis of the first 15 minutes of the transient. It is expected that "batch mode" computation is sufficient for the latter part of the transients.

Semiscale

Semiscale test S-FS-6 was successfully run during the week ending June 29, 1985. This is a 15 percent break simulation of the steam generator feedwater line of a Combustion Engineering (CE) System 80 type PWR. The test consisted of three parts. During the first 600 seconds of the transient, only automatic systems were operated to mitigate the accident consequences. Thereafter, the test operators used procedures representative of commercial PWR procedures, e.g., primary charging and letdown, pressurizer heater operation and steam generator auxiliary feedwater, to stabilize the plant such that plant cooldown could begin. Thereafter, the broken loop steam generator was isolated and reflooded with auxiliary feedwater flow of 25 percent scaled flow in order to study heat transfer characteristics as a function of steam generator secondary side liquid level. The entire transient lasted approximately 6 hours. The test was performed 9 days ahead of the scheduled test date of July 5, 1985.

Semiscale report EGG-SEMI-6884, "Quick Look Report for Semiscale MOD-2C Experiments S-LH-1 and S-LH-2", was received this week. Both tests simulated 5 percent cold leg breaks in which the phenomenon of core level depression prior to loop seal clearance was being evaluated. Tests LH-1 and LH-2 were run with a downcomer-to-upper-head bypass flow of 0.9 percent and 3.0 percent of core flow respectively. This approximates the conditions of Semiscale tests S-UT-8 and S-UT-6 which were performed on a different Semiscale design in 1981; the results of the two sets of tests were also comparable i.e., core level depression was observed before loop seal clearing in both cases, but this was accompanied by core heating only for the case of lower bypass flow (S-UT-8 and S-LH-1).

This depression is the result of a liquid holdup in the hot legs and up flow portion of steam generator tubes causing a manometer-type depression of core liquid level. For LH-1 the minimum collapsed core liquid level was about 110 cm. above the bottom of the core and for LH-2 about 230 cm.

The Quick Look Report examines the available data for these tests in detail in order to provide phenomenological explanations for the observed behavior. For example the report concludes that loop seal clearing is a smooth continuous process rather than a violent blowout. Multidimensional core thermal response was observed that suggest falling liquid films on the heater rods. Core liquid froth levels were also derived from core density measurements and this froth level was used to explain initial heatup in S-LH-1 to 663°F before loop seal clearing, but no heatup in S-LH-2. Subsequent heatup of the rods before accumulator flow, as a result of continued loss of system coolant inventory, was 915°F for S-LH-1 and less than that for S-LH-2.

TRAC-PWR Users Guide

The Los Alamos National Laboratory (LANL) has provided the NRC with a draft "TRAC Users Guide". The Users Guide was developed in order to provide non-LANL users with a road map of how to set up both transient and steady state input models from raw data.

It is also designed to document the user experience gained by both LANL and other U.S. labs in running the code. This document, which will be published as a NUREG, is considered to be a companion document to the TRAC-PF1/MOD1 code manual and should be helpful to our domestic code users, as well as our international partners in the International Code Assessment Program (ICAP).

Multi-Loop Integral System Test (MIST)

The MIST testing program is designed to provide thermal hydraulic data related to small break (LOCA) transients to benchmark Babcock and Wilcox (B&W) analysis models. The pressure component code hydro testing of the MIST facility was successfully conducted on June 24, 1985. The hydro testing encompassed the hot and cold leg piping, the upper reactor vessel, downcomer, pressurizer, steam generators, and secondary system.

ENCLOSURE E

University of Maryland 2x4 B&W Simulation Loop

Construction of the University of Maryland 2x4 Babcock & Wilcox (B&W) simulation loop is essentially complete. This is a small-scale 2x4 B&W facility which is part of the Integral System Test (IST) program that will provide integral system test data to address scaling compromises of sealed experimental facilities. Cold hydro test to 527 psia has been accomplished, and hot testing is underway. The loop has been pressurized to its operating pressure of 300 psia, and has reached a temperature of 300°F. Most instrumentation is installed and computer software is being developed for data collection. Initial hot tests have included steady state natural circulation runs as well as small break (LOCA) blowdowns to develop procedures to operate the loop.

Quantifying the Uncertainty of Best-Estimate Codes TRAC-PF1/MOD1, RELAP5/MOD2 and Trac-BF1

A meeting was held in Idaho Falls, Idaho to discuss the quantification of code uncertainty of the best-estimate thermal hydraulic codes RELAP5/MOD2, TRAC-PF1/MOD1 and TRAC-BF1. NRC contractor representatives responsible for code assessment attended from Brookhaven National Laboratory (BNL), INEL, Sandia National Laboratory, and LANL, as well as the responsible NRC program managers. Methodology for assessing the codes by comparing code predictions with experimental data were discussed with a view to reaching a consensus approach. The assessment effort includes the International Code Assessment and Applications Program, domestic independent assessment and assessment through use of the codes in experimental programs, including 2D/3D, ROSA-IV, MIST/OTIS, and Semiscale. The procedures include statement of the uncertainty in terms of a limited set of key parameters combining the results of numerous assessment cases to form an assessment data base. The meeting provided general agreement on procedures and provided guidance on statistical methods that may be used to provide code uncertainty statements expressed in terms of standard deviation.

Electrical Penetration Assembly - Severe Accident Test

A D.G. O'Brien containment electrical penetration assembly, identical to those used in many nuclear plants, was tested at Sandia National Laboratory for leakage and functional electrical behavior when exposed over a 10-day period to a simulated PWR severe accident environment of 360° F. and 150 PSIA. These environmental parameters envelope the conditions calculated to develop inside containment for the dominant PWR severe accident scenarios involving core melt and vessel failure. Given that research to define severe accident source terms has not been completed, the electrical penetration assembly was irradiated to 200 Mrad, representing the equipment accident dose based on current regulatory practice and thermally aged to the equivalent of a 40-year life prior to the test. A negligible increase in steam leakage through the electrical penetration assembly was observed, indicating that containment integrity would have been maintained. However, the electrical resistance of the individual conductors and qualified electrical cable connections went essentially to zero by the second day of the test and never recovered. On disassembly moisture was found in the electrical connector and glass to metal seal interface indicating that short circuits caused by moisture ingress through the seals was the likely cause of electrical failures. Tests of other currently used electrical penetration assembly designs to BWR severe accident conditions are planned to begin in September.

ENCLOSURE E

JUL 5 1985

ITEMS OF INTEREST
OFFICE OF INTERNATIONAL PROGRAMS
WEEK ENDING JULY 7, 1985

International Meetings

The following international meeting notices supplement those announced earlier:

September 9-10, 1985, Cadarache, France - IAEA Consultants Meeting on the Environmental Behaviour of Radium

October 14-18, 1985, Portoroz, Yugoslavia - Technical Committee/Workshop on the IAEA Program on Computer-Aided Safety Analysis

February 2-6, 1986, San Diego, California, USA - IAEA International Topical Meeting on Thermal Reactor Safety

IAEA Vacancy Notices

The International Atomic Energy Agency has announced the following vacancy notices, which are posted on NRC bulletin boards, for positions at the IAEA headquarters in Vienna, Austria:

- P-4 Containment and Surveillance Specialist; Section for Development of Instruments, Methods and Techniques; Division of Development and Technical Support; Department of Safeguards
- P-5 Nuclear/Chemical/Environmental Engineer, Waste Management Section, Division of Nuclear Fuel Cycle, Department of Nuclear Energy
- P-2 Development Programmer, Computer Section, Division of Scientific and Technical Information, Department of Nuclear Energy and Safety
- P-4 Senior Systems Analyst, Computer Section, Division of Scientific and Technical Information, Department of Nuclear Energy and Safety
- P-3 Management and Organizational Analyst, Office of Internal Audit and Management Services, Department of Administration
- P-4 Personnel Management Specialist; Section for Administrative Support; Division of Standardization, Training and Administrative Support; Department of Safeguards

International Visitors

On Wednesday Mr. Paul Woodhouse, Principal Inspector of the UK Nuclear Installations Inspectorate, met with RES and NRR representatives to discuss the current status of NRC's examination of the station blackout, electrical equipment reliability, and equipment aging issues.

JUL 5 1985

ENCLOSURE G

OFFICE OF STATE PROGRAMS

ITEMS OF INTEREST

WEEK ENDING JULY 5, 1985

LOW-LEVEL WASTE COMPACTS

The Appalachian Compact bill has cleared the Pennsylvania Senate and been sent to the House.

Although it had appeared that New York State executive and legislative officials were near agreement on a low-level waste management bill, it was not enacted before the summer legislative recess. The New York Legislature reconvenes in October.

ENCLOSURE H

JUL 5 1985

OFFICE FOR ANALYSIS AND EVALUATION OF OPERATIONAL DATA

ITEMS OF INTEREST

WEEK ENDING JULY 5, 1985

Final Trends and Patterns Report on Feedwater Transients at Westinghouse Plants (AEOD/P405)

AEOD issued the final report on trends and patterns of feedwater transients at Westinghouse PWRs (AEOD/P405). This report is supplemented by Engineering Evaluation Report AEOD/E418.

This study, based upon available information, was initiated to characterize the incidence of feedwater transients and, if possible, pinpoint the causes. The report covers January 1981 through June 1983.

It was concluded that 2-loop and early 4-loop Westinghouse plants share a transient rate which is a factor of 10 lower than that for 3-loop and later 4-loop units. In addition, we found that Salem 2 was a statistical outlier due to its high transient rate. We recommended that Region I continue to monitor planned improvements to the feedwater and condensate systems at Salem to assure they are effective.

AEOD Studies in Progress

A listing of planned and ongoing studies within AEOD was transmitted to the Program Office Directors and the Regional Administrators. This listing is issued bimonthly and includes activities in the Reactor Operations Analysis Branch, the Program Technology Branch, and the Nonreactor Assessment Staff. A total of 61 studies are planned or in progress. Six studies were completed and issued during the past two months. In addition, one study, "Safety Implications Associated with In-Plant Pressurized Gas Storage and Distribution Systems in Nuclear Power Plants," was completed by our contractor (ORNL) and published as NUREG/CR-3551.

JUL 5 1985

ENCLOSURE J

ITEMS ADDRESSED BY THE COMMISSION - WEEK ENDING JULY 5, 1985

- A. STAFF REQUIREMENTS - DISCUSSION/POSSIBLE VOTE ON FINAL RULE ON BACKFITTING, 2:00 P.M., WEDNESDAY, JUNE 26, 1985, COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC ATTENDANCE) Memo SECY to J. Tourtellotte dated 7/3/85

The Commission met to discuss the final backfit rule (SECY-85-222).

Chairman Palladino requested Commissioners to give prompt attention to the final rule and provide their notation votes to the Office of the Secretary.
(OCM) (SECY Suspense: 7/10/85)

- B. STAFF REQUIREMENTS - DISCUSSION/POSSIBLE VOTE ON REVIEW OF ALAB-800 AND RELATED MATTERS (SHOREHAM), 4:00 P.M., TUESDAY, JUNE 11, 1985 COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (PORTION OPEN TO PUBLIC ATTENDANCE) Memo SECY to RECORD dated 7/3/85

The Commission met to vote on the disposition to be given to the Commission's review of ALAB-800 with regard to whether or not the alternate AC power system should be treated as vital equipment at Shoreham.

At the beginning of this meeting Chairman Palladino announced the results of Commission votes in the immediately preceding closed session to: (1) deny the request of Newsday to open the closed meeting deliberations to public attendance and (2) deny the request of Suffolk County (filed by Herbert H. Brown of the law firm of Kirkpatrick and Lockhart) to postpone the Commission meeting on ALAB-800.

The Commission voted 3-2 (with Commissioners Roberts and Zech disagreeing) to affirm the decision of the Appeal Board in ALAB-800 insofar as it finds that at least a portion of the alternate emergency power supply should be treated as vital equipment and protected as such under the Commission's safeguards regulations. The Commission further agrees, as recommended by its staff, that it is sufficient for the EMD diesel unit alone to be treated as vital equipment. The Commission directs the Licensing Board to proceed in accordance with the Commission's opinion. The effect of this decision is that for the present, no low power authorization is granted for Phases III and IV of low power testing.

The Commission instructed OGC and OPE to prepare an order for Commission review which implements the Commission's decision.
(OGC/OPE) (SECY Suspense: 7/29/85)

C. STAFF REQUIREMENTS - AFFIRMATION/DISCUSSION AND VOTE, 1:00 P.M., THURSDAY, JUNE 27, 1985, COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC ATTENDANCE) Memo SECY to W. J. Dircks dated 7/3/85

I. SECY-85-109 - Amendments to 10 CFR Part 60 -- Disposal of High-Level Radioactive Wastes in Geologic Repositories

The Commission, by a 5-0 vote, approved for publication final amendments to 10 CFR Part 60 which contain criteria for geologic disposal of high-level radioactive wastes in the unsaturated zone.

You should forward the Federal Register Notice for signature and publication in the Federal Register.

(EDO/RES)

(SECY Suspense: 7/16/85)

II. Severe Accident Policy Statement

The Commission, by a 4-1 vote (with Commissioner Asselstine disapproving), approved the severe accident policy statement as modified in the attached copy (Attachment 1).

The Commission, by a 5-0 vote, concurred with Commissioner Bernthal's position that "Omitting the word 'property' from the ... Severe Accident Policy Statement in no way changes Commission policy and precedent that off-site property value is to be taken into consideration in cost-benefit and probabilistic risk assessment analysis."

NUREG-1070 should be modified as necessary to conform to the revised policy statement and the above statement of the Commission on off-site property value. (NRR)

The Commission requests that the staff prepare a paper on the bases for quantifying, in PRA type cost-benefit analysis, the values and related probabilities of possible offsite property losses. (NRR)

Chairman Palladino's and Commissioner Bernthal's memos on the subject of offsite property are attached for the record (Attachments 2 and 3).

Commissioner Asselstine indicated that he will have separate views which will be available within four (4) working days.

You should forward a copy of this policy statement revised as noted for signature and publication in the Federal Register.

(EDO) (NRR)

(SECY Suspense: 7/19/85)

JUL 5 1985

ENCLOSURE 0

D. STAFF REQUIREMENTS - STAFF BRIEFING ON FINAL RULE ON HEU REGULATIONS FOR DOMESTIC NON-POWER REACTORS, 2:00 P.M., WEDNESDAY, JUNE 19, 1985, COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC ATTENDANCE) Memo SECY to W. J. Dircks and H. H. E. Plaine dated 7/5/85

The Commission met to be briefed by staff on a proposed rule to limit the use of highly enriched uranium (HEU) in research and test reactors (SECY-85-213).

The Commission requested staff to provide the following:

1. an order directing licensees to remove excess unirradiated fuel (providing for no disruption of normal operations);
(NMSS/OGC) (SECY Suspense: 7/19/85)
2. a revised final rule on conversion from HEU to LEU reflecting discussion on value impact analysis and including a generic statement with regard to avoiding litigation for any necessary license amendments by using generic rulemaking; and
(RES/NMSS/OGC) (SECY Suspense: 7/19/85)
3. a letter to Congress requesting federal funding to finance the conversion.
(NMSS) (SECY Suspense: 7/19/85)

The Commission requested the Office of the General Counsel to provide an analysis of options, including generic rulemaking to avoid the need for adjudicatory hearings on license amendments involving conversion from HEU to LEU in domestic non-power reactors.

(OGC) (SECY Suspense: 7/19/85)

E. STAFF REQUIREMENTS - DISCUSSION OF COMMISSION POSITION ON PRICE-ANDERSON, 2:00 P.M., THURSDAY, JUNE 20, 1985, COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC ATTENDANCE) Memo SECY to RECORD dated 7/5/85

The Commission met to receive testimony on the renewal of the Price-Anderson Act. Representatives of the nuclear industry, interest groups, insurance groups, the State of Wisconsin and the staff provided views on the Price-Anderson Act for Commission consideration.

Chairman Palladino requested that each Commissioner submit to the Secretary within two working days of the meeting date a statement indicating support or non-support for some or all of the recommendations in the Commission's 1983 report to Congress on Price-Anderson.

(Subsequently, on June 24, 1985 each Commissioner provided a copy of his views on the 1983 report to the Office of the Secretary.)

E. CONTINUED

The following persons made presentations to the Commission:

Steve C. Griffith, Jr.
Senior Vice President and General Counsel
Duke Power

George Gleason
American Nuclear Energy Counsel

William R. Bechtel, Director
Federal-State Relations Office
State of Wisconsin

Keiki Kehoe
Environmental Policy Institute

Kathleen Welch
U.S. Public Interest Research Group

Michael Faden
Union of Concern Scientists

Richard Schmalz
American Nuclear Insurers

Joseph Marrone
Vice President and General Counsel
American Nuclear Insurers

Jerry Salzman
NRC Staff, State Programs

Guy Cunningham
NRC Staff, Executive Legal Director

There were no requirements for the staff at this meeting.

JUL 5 1985

NRR MEETING NOTICES*

JULY 5, 1985

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>APPLICANT/ ATTENDEES</u>	<u>NRR CONTACT</u>
7/8-12/85 8:00 am	50-410	Nine Mile Pt. Site Scriba, NY	Meeting of Equipment Seismic Qualification Review Team and Pump and Valve Operability Review Team	Niagara Mohawk Power	M. Haughey
7/9/85 8:00 am	50-309	MNBB-6110 Bethesda	To discuss Maine Yankee's Detailed Control Room Design Review	Maine Yankee Atomic	P. Sears
7/9/85 10:00 am	50-247	Region I Office King of Prussia, PA	To discuss Technical Specifi- cation amendment application	Consolidated Edison	J. Neighbors
7/9-12/85	50-424 50-425	Vogtle Site Waynesboro, Ga.	In-Progress Audit of Unit 1 Control Room	Georgia Power Co. Southern Co. Serv. Lawrence Livermore Lab.	M. Miller
7/10/85 10:00 am	50-261	Room 438 (Phillips) Bethesda	To discuss status of multi- plant, TMI and plant specific open items	Carolina Power & Light	G. Requa
7/11/85 1:30 pm	50-250 50-251	P-114 Bethesda	To discuss Turkey Point Tech Specs	Florida Power & Light	D. McDonald

PLEASE BE ADVISED THAT DAILY RECORDING CAN BE HEARD ON 492-7166

*Copies of summaries of these meetings will be made publicly available and placed in the respective docket file(s) in the NRC and local public document rooms.

ENCLOSURE P

JUL 5 1985

NRR MEETING NOTICES*

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>APPLICANT/ ATTENDEES</u>	<u>NRR CONTACT</u>
7/16/85 10:00 am	50-271	P-110 Bethesda	To discuss Fire Protection	Vermont Yankee	R. Hermann
7/16/85 9:00 am		P-422 Bethesda	To discuss B&W Owners Group comments re draft NUREG-0844 on staff's proposed technical resolution of steam generator USIs	B&W SG Owners Group	E. Murphy
7/17/85 10:00 am	50-244	P-114 Bethesda	To discuss NRC staff's recommen- dation of neutron fluence to be used in Surveillance Capsule T calculations in assessing radi- ation damage to the reactor vessel	Rochester Gas & Elec. (Ginna)	C. Miller
7/17/85 10:00 am	50-321 50-366	Region II Atlanta, Ga.	To discuss GPC plans related to Hatch 1&2 and the status of licensing activities	Georgia Power Co.	G. Rivenbark

PLEASE BE ADVISED THAT DAILY RECORDING CAN BE HEARD ON 492-7166

*Copies of summaries of these meetings will be made publicly available and placed in the respective docket file(s) in the NRC and local public document rooms.

ENCLOSURE P

NMSS MEETING NOTICES

FOR WEEK ENDING: 7/5/85

Division of Fuel Cycle and Material Safety

JUL 5 1985

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>ATTENDEES/APPLICANT</u>	<u>NRC CONTACT</u>
7/10/85 9:00 a.m.	70-2	Room 517 Willste	To discuss license application for an independent spent fuel storage installation at VEPCO Surry.	M. Smith, et al., (VEPCO) J. Roberts (FC) F. Sturz (FC) L. Rouse (FC)	Roberts
7/17-18/85	70-8	West Jefferson, OH Columbus, OH	Meetings with Battelle and Ohio EPA to discuss environmental assessment of low-level waste incineration.	H. Toy, et al., (Battelle) J. McCoy, et al., (Ohio EPA) P. Loysen (FC) J. Ayer (FC)	Loysen
7/17/85 9:00 a.m.	72-3	Room 517 Willste	To discuss license application for an independent spent fuel storage installation at H. B. Robinson 2.	Reps of CP&L J. Roberts (FC) F. Sturz (FC)	Roberts
7/23/85 2:00 p.m.		Albuquerque, NM	Participate in N14 Management Committee Meeting.	C. MacDonald (FC)	MacDonald
7/24/85		Chicago, IL	Attend ASME Task Group Meeting.	C. MacDonald (FC)	MacDonald

Division of Safeguards

7/23/85 9:00am	Silver Spring, Md.	SG Interface Meeting	NMSS, IE, NRR RES, & Regions	Baker
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Division of Waste Management

None

ENCLOSURE P

June 7, 1985

JUL

5 1985

RII MEETING NOTICE

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>ATTENDEES/ APPLICANT</u>	<u>NRC CONTACT</u>
07/12/85 10:00 am		RII Office	Enforcement Conference - Duke Power Company - McGuire facility	Licensee, Regional Administrator and selected RII staff members	Walker

ENCLOSURE P