

DMB

JUL 19 1985

Docket: 50-267

Mr. O. R. Lee, Vice President  
Electric Production  
Public Service Company of Colorado  
P.O. Box 840  
Denver, Colorado 80201

Dear Mr. Lee:

We have reviewed the various submittals you have made in response to our October 16, 1984, "Preliminary Report Related to the Restart and Continued Operation of the Fort St. Vrain Nuclear Generating Station," and other submittals related to subjects which have required resolution prior to the resumption of operations at Ft. St. Vrain (FSV). We have provided the results of our reviews of the various areas of concern in a number of recently issued Safety Evaluations (SEs). These safety evaluations contain the NRC findings on all issues associated with the above report. In addition, we have approved some of your proposals for resolving some of our concerns which will require long-term actions. A listing of our SEs and approvals is contained in Enclosure 1, a listing of your various commitments is contained in Enclosure 2.

Our June 26, 1984, confirmatory action letter (CAL) confirmed your commitment that FSV would be maintained in a shutdown condition until the NRC authorized a different status. Our reviews have now progressed to such a point that we have determined that you have satisfied the commitments contained in our June 26, 1984, letter and have addressed the issues raised in the October 16, 1984, report. We find it acceptable, therefore, for FSV to be operated in a "dry-out" mode to aid in the removal of moisture. Such an operation would hold reactor power below a level at which boilout in the economizer-evaporator-superheater section of the steam generator would be achieved, but in no case greater than 15% Rated Thermal Power until such time as certain equipment qualification questions are resolved. Our evaluation of the acceptability of operating FSV in this mode is contained in the enclosed Safety Evaluation (Enclosure 3).

In addition, we have reviewed your proposed compensatory measures in the area of fire protection, contained in your letter dated July 11, 1985, (P-85245) and find them to be an acceptable basis to allow interim operations until our technical review is completed. We have confirmed your commitments in the listing contained in Enclosure 2; a SE of our finding will be provided under separate cover.

In authorizing the restart and the operation of Fort St. Vrain at power levels not to exceed 15% power, we understand that you have committed to the following actions:

RIV:RSB/ES  
REIreland/lk  
7/19/85

RPB/  
JPJaudon  
7/19/85

RPB  
EHJohnson  
7/19/85

DRSP  
RPDenise  
7/19/85

RA  
RDMartin  
7/19/85

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PDR ADOCK 05000267  
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1. Unless authorized by NRC to proceed to a higher power level, PSC will restrict plant power level to no greater than 15% power. PSC shall notify the NRC of the completion of aging and equipment operability studies for equipment qualification under 10 CFR 50.49.
2. PSC will continue to implement those items listed as complete and will resolve the longer range issues contained in Enclosure 2. In doing so, the scope and schedule for resolution of these issues, that has been previously agreed to with the NRC staff, shall not be altered without prior agreement of the NRC staff.

If your understanding of these commitments is not the same as ours, please contact our office within 24 hours at (817) 860-8100.

Since these reporting requirements relate solely to the FSV, OMB clearance is not required under P.L. 96-511.

Sincerely,

Original signed by  
Robert D. Martin

Robert D. Martin  
Regional Administrator

Enclosures:

1. List of SEs and Approvals
2. List of PSC Commitments
3. SE Related to Environmental Qualifications

cc:

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Public Service Company of Colorado  
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Denver, Colorado 80201

Mr. David Alberstein, 14/159A  
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(cont. on next page)

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Platteville, Colorado 80651

L. Singleton, Manager, Quality  
Assurance Division  
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Colorado Radiation Control Program Director

bcc distrib. by RIV:

RSB Reading File

R. E. Ireland, RSB/ES

E. Haycraft, DRSP/LA

Butcher, ORB3

G. Lainas, DL

T. King, NRR

Resident Inspector

P. Wagner, RSB/ES

RIV Official Reading File

CPB/NRR

K. Heitner, ORB3

R. D. Martin, RA

R. Denise, DRS&P

E. Jordan, IE E.

D. Eisenhut, D/DL

J. Taylor, IE

LIST OF SAFETY EVALUATION/APPROVALS ISSUED  
RELATED TO THE RESTART OF FORT ST. VRAIN

<u>Date</u>	<u>Subject</u>
July 8, 1985	Prestressed Concrete Reactor Vessel Tendon Wire Corrosion Problem SE
July 9, 1985	Liquid Effluent Release from the Reactor Building Sump.
July 10, 1985	Emergency Electrical Power Systems SE
July 12, 1985	Assessment Report SE (CRDM, Technical Specifications, and Management Controls)
July 12, 1985	CRDM Bearing SE letter

PUBLIC SERVICE COMPANY OF COLORADO COMMITMENTS  
RELATED TO THE OPERATION OF THE  
FORT ST. VRAIN NUCLEAR GENERATING STATION

<u>Item</u>	<u>Commitment Description/Requirement (Reference)</u>	<u>Schedule</u>
1.	Implement a program to maintain CRDM temperature within acceptable limits. (P-185199 and P-85242)	Complete (see Item 8)
2.	Provide an improved CRDM Surveillance and preventative maintenance program (P-85199 and P-85242), which includes consideration of the new bearing performance (P-85201).	6 months after restart
3.	Implement a procedure to prevent overdriving of the CRDM. (P-85040) (see SOP 12-01, Issue 15)	Complete
4.	Implement a procedure to require a reactor shutdown under conditions where CRDM purge flow is lost or when high levels of moisture exists in the coolant. (P-85040)	Complete (see Item 8)
5.	Monitor CRDM temperatures. (P-85199 and P-85242)	Complete (see Item 8)
6.	Provide status reports/progress reports on the Nuclear Performance Enhancement Program on a quarterly basis. (P-85217)	Approximately every 3 months
7.	Implement the Technical Specification (TS) Upgrade Program (P-85098)	
	a. Provide a final draft proposal of the Upgraded TSs for NRC comment. (P-85243)	10/15/85
	b. Submit a license amendment to incorporate the Upgraded TSs. (P-85243)	90 days after NRC comments on Item 7a
	c. Implement the Upgraded TSs. (P-85243)	Approximately the 4th Refueling Outage (License Amend- ment will define)

<u>Item</u>	<u>Commitment Description/Requirement (Reference)</u>	<u>Schedule</u>
8.	Improve control rod and reserve shutdown reliability.	
	a. Propose Technical Specification changes. (P-85242)	Complete
	b. Implement proposed requirements through interim procedures. (P-85242)	Complete
	c. Incorporate the proposed Technical Specifications in the Upgrade Program.	(See Item 7)
9.	Develop a plan to implement approved modifications to control moisture ingress and submit annual reports on progress. (P-85022 and P-85082)	Annually Report Progress
10.	Control moisture in CRDM purge system. (P-85032)	4th Refueling Outage
11.	Perform environmental requalification testing of a CRDM assembly (P-85032) including the temperature sensor epoxy. (P-85195)	12/30/85
12.	Refine the "watt-meter" test for verification of control rod full insertion or develop an alternative test. (P-85040 and P-85199)	6 months after restart (see Item 2)
13.	Investigate a design change to provide a positive stop to prevent CRDM overtravel; report results to the NRC. (P-85003)	1/1/86
14.	Conduct an integrated systems study to resolve control rod position indication, maintenance and operability questions. (P-85003)	6/30/86
15.	Implement QA Procedure Review Program. (P-85028)	7/1/86
16.	Liquid Effluent Releases from the Reactor Building Sump	
	a. Implement procedures to perform all effluent releases from the reactor building sump in batch mode. (P-85212)	Complete
	b. Investigate installing in-line, beta sensitive, effluent monitors and report on progress. (P-85212)	In progress



<u>Item</u>	<u>Commitment Description/Requirement (Reference)</u>	<u>Schedule</u>
17.	PCRV Tendon Surveillance Requirements	
	a. Incorporate the revised Tendon Surveillance Program into the Technical Specifications. (P-85199)	(See Item 7)
	b. Implement the revised program through the use of interim procedures. (P-85199)	Complete
	c. Provide the results of the revised program to the NRC	Approximately Every 6 months
18.	Propose modifications in the Emergency Diesel Generator circuit breakers' control circuitry to resolve the independence problem. (P-85208)	9/15/85
19.	Fire Protection Interim Requirements	
	a. Implement the special (and interim) repair and operating procedures. (P-85113 and P-85245)	Complete
	b. Implement a fire watch program. (P-85245)	Complete
20.	Complete the CRDM bearing performance testing and incorporate results into the improved CRDM surveillance and preventative maintenance program identified in Item 2 above. (P-85201)	Report on progress quarterly