

## LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0104  
EXPIRES - 8/31/85

FACILITY NAME (1)										DOCKET NUMBER (2)										PAGE (3)																																							
Peach Bottom Atomic Power Station - Unit 2										0 5 0 0 0 2 7 7 1										OF 0 3																																							
TITLE (4)																																																											
Scram and Group II/III Isolations on Reactor Low Level																																																											
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																													
MONTH			DAY			YEAR				YEAR			SEQUENTIAL NUMBER			REVISION NUMBER				MONTH			DAY			YEAR				FACILITY NAME										DOCKET NUMBER (8)																			
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THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																																																											
OPERATING MODE (9)										20.402(b)										20.406(c)										X 60.736(12)(iv)										73.716(i)																			
POWER LEVEL (10)										0 0 0										20.406(a)(1)(i)(ii)										60.346(11)										60.736(12)(v)										73.716(j)									
										20.406(a)(1)(i)(iii)										60.346(12)										60.736(12)(vi)										OTHER (Specify in Abstract below and in Test, NRC Form 364-A)																			
										20.406(a)(1)(i)(iv)										60.736(12)(vii)										60.736(12)(viii)(A)																													
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LICENSEE CONTACT FOR THIS LER (12)																																																											
NAME																				TELEPHONE NUMBER																																							
W. C. Birely, Senior Engineer, Licensing Section																				2 15 8 4 1 7 5 0 4 8																																							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																											
CAUSE			SYSTEM			COMPONENT				MANUFACTURER			REPORTABLE TO NRC				CAUSE			SYSTEM			COMPONENT				MANUFACTURER			REPORTABLE TO NRC																													
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SUPPLEMENTAL REPORT EXPECTED (14)																																																											
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO										EXPECTED SUBMISSION DATE (15)																													
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ABSTRACT (Limit to 1600 words, i.e., approximately fifteen single-spaced typewritten lines) (16)

Abstract: 2-85-14

On August 20, 1985 with Unit 2 in the shutdown mode, a scram signal and a Group II and Group III isolation occurred due to a reactor low water level signal. Because Unit 2 was shutdown, no control rod movement occurred. The "C" reactor feedwater pump startup bypass valve was being used to automatically control reactor water level. Oscillations in the level had been occurring which were not adequately controlled by the operator and lead to the scram. Following the scram and isolations, the bypass valve was used manually to recover and control reactor level. The controller was later tuned to provide better response and was demonstrated to be functioning properly on August 24, 1985. The reactor operator was counselled concerning the proper operator response to reactor water level transients.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/86

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					0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 364a (17))

Description of the Event:

On August 20, 1985, at 0225 hours, the Reactor Protection System initiated a scram signal and the Primary Containment Isolation System initiated Group II and Group III isolations due to a reactor low water level signal. Unit 2 had been placed in the shutdown mode at 2319 on August 19, 1985 to begin a maintenance outage; therefore, no control rod movement occurred.

The reactor pressure was decreasing and was approximately 300 psia at the time of the event. The "C" feedwater pump startup bypass valve, A0-8091, was being used to automatically control reactor water level. The bypass valve controller, LIC-8091, was responding slowly causing swings in reactor water level.

During one of the swings in level, the level dropped below the low level alarm point. The reactor operator switched the level controller from auto control to manual control and manually increased the opening of the valve to restore the level. The level started to increase and the operator assumed that the level would continue to increase. At this time, the operator's attention was directed to another operational task. The reactor level had not yet increased enough to clear the low level alarm. The level again started to decrease and reached the low level scram setpoint. The level was manually recovered with the bypass valve.

All of the appropriate valves closed in response to the Group II and Group III isolation signals. The scram and isolation signals were reset.

The EIIS code for the affected system, Feedwater, is SJ and for the affected component, level indicating controller, LIC.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO. 3180-0104

EXPIRES 8/31/86

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TEXT (if more space is required, use additional NRC Form 366A (1))

Consequences of the Event:

All control rods were in the full-in position and the reactor was subcritical. Reactor water level did not decrease below the low level scram setpoint. Adequate coolant was available from other operable components of the condensate and feedwater systems. The safety significance of this event is insignificant because cooling of the reactor was not in jeopardy.

Cause of the Event:

The cause of this event was low reactor water level. The low water level was caused by the slow response of the LIC-8091 controller and the failure of the reactor operator to provide the proper attention needed during the swings in level. The slow response of the controller when in the automatic mode had caused the level to swing for approximately one hour prior to the event. Even though the level had started to increase just prior to the event, the operator should not have redirected his attention to other operational tasks until the level had been properly restored to normal.

Corrective Actions:

Following the scram, level was recovered and the appropriate signals reset. To prevent recurrence, the LIC-8091 controller was tuned to increase its response. The controller was tested on August 24, 1985 and operation was satisfactory. The reactor operator was counseled to be more thorough and timely when monitoring reactor parameters.

Previous Similar Occurrences:

None.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

September 24, 1985

Docket No. 50-277

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Washington, DC 20555

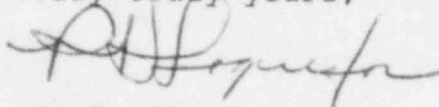
SUBJECT: Licensee Event Report  
Peach Bottom Atomic Power Station - Unit 2

This LER deals with a scram signal and Group II and Group III isolations which occurred while Unit 2 was shutdown.

Reference:	Docket No. 50-277
Report Number:	2-85-14
Revision Number:	00
Event Date:	August 20, 1985
Report Date:	September 24, 1985
Facility:	Peach Bottom Atomic Power Station RD 1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv). We regret the delayed submittal of this report and any inconvenience it may have caused.

Very truly yours,



W. T. Ullrich  
Superintendent  
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC  
T. P. Johnson, Resident Inspector

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