

NRC Form 288
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/86

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Grand Gulf Nuclear Station - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 4 1 6				PAGE (3) 1 OF 0 2	
TITLE (4) Reactor Water Cleanup Isolations															
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES N/A				DOCKET NUMBER(S) 0 5 0 0 0		
0 6	2 3	8 5	8 5	0 2 3	0 0	0 7	1 8	8 5					0 5 0 0 0		
OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)													
POWER LEVEL (10) 0 9 5		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(1)(iv)				73.71(b)	
		20.405(a)(1)(i)				50.38(e)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)	
		20.405(a)(1)(ii)				50.38(e)(2)				<input type="checkbox"/> 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)	
		20.405(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)					
20.405(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(ix)							
LICENSEE CONTACT FOR THIS LER (12)															
NAME Angela H. Horton/Licensing Engineer										TELEPHONE NUMBER 6 0 1 4 3 7 1 - 12 1 4 9					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC					
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 23 and June 26, 1985, a relay internal to a temperature switch failed causing an Reactor Water Cleanup (RWC) System isolation and a Division 2 Main Steam Isolation Valve isolation signal. The June 23 isolations were reset within 15 minutes. The isolations that occurred on June 26 were reset but 5 minutes later the isolations reoccurred and were reset within 7 hours.

The isolations on June 26 were due to a failed relay in the 86VTFF-EG-4844 Riley Panalarm switch N605B. The specific cause of the June 23 isolations could not be determined but it is suspected the isolations were caused by the same switch. Switch N605B was replaced. MP&L is conducting an investigation to determine corrective actions needed to eliminate spurious isolations caused by the Leak Detection System (E31).

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NRC Form 366A
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		OF	
Grand Gulf Nuclear Station	0 5 0 0 0 4 1 6	8 5	0 2 3	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of Reportable Occurrence

On June 23 at 2322 and on June 26 at 1135 the Reactor Water Cleanup (RWCU) System isolated.

Initial Conditions

The plant was operating at approximately 95 percent reactor power.

Status of Redundant or Backup Systems

Not applicable.

Nature of Occurrence

On June 23 and on June 26 a Main Steam Line (MSL) temperature switch tripped resulting in an RWCU isolation and a Division 2 Main Steam Isolation Valve isolation signal.

Immediate Corrective Action Taken

The June 23 isolations were reset within 15 minutes. The isolations that occurred on June 26 were reset but 5 minutes later the isolations reoccurred and were reset within 7 hours.

Apparent Cause

The isolations on June 26 were due to a failed relay in the 86VTFF-EG-4844 Riley Panalarm switch N605B. The specific cause of the June 23 isolations could not be determined but it is suspected the isolations were caused by the same switch.

Supplemental Corrective Action

Switch N605B was replaced. MP&L is conducting an investigation to determine corrective actions needed to eliminate spurious isolations caused by the Leak Detection System (E31).

Safety Assessment

The isolation of RWCU does not effect plant safety or power generation.

DOCUMENT SCREENING RECORD SHEET

Document Screened: LER 85-023-0 (AECM-85/0216)

Condition being reviewed: Reactor Water Cleanup Isolations

Conclusion:

- ☐ Deviation requiring evaluation (10CFR21)
☐ Deficiency requiring evaluation (10CFR50.55(e))
☒ No further evaluation required

Remarks: On June 23 and 26 a Main Steam Line temperature switch tripped resulting in an RWCU isolation and a Division 2 Main Steam Isolation Valve isolation signal. The June 23 isolations were reset within 15 minutes. The June 26 isolations were reset but 5 minutes later reoccurred and were reset within 7 hours. The June 26 isolations were caused by a failed relay in the 86VTFF-EG-4844 Riley Panalarm switch N605B. The specific cause of the June 23 isolations could not be determined but is believed to have been the same switch. Switch N605B was replaced. MP4L is conducting an investigation to determine corrective actions needed to eliminate spurious isolations caused by the leak detection system (E31). The isolation of RWCU does not effect plant safety or power generation.

Screened by: E. Boyd Hyatt
Date: 6/18/85

by: Lorrell Hughes
7/14/85



SCREENING FOR DEVIATIONS AND DEFICIENCIES
ATTACHMENT I to Procedure 1.7

REV. 0

DATE
OCT 24 1984

PAGE 1 of 1



MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

P. O. BOX 1640, JACKSON, MISSISSIPPI 39215-1640

July 18, 1985

NUCLEAR LICENSING & SAFETY DEPARTMENT

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-29
File: 0260/L-835.0
Reactor Water Cleanup Isolations
LER 85-023-0
AECM-85/0216

Attached is Licensee Event Report (LER) 85-023-0 which is a final report.

Yours truly,

L. F. Dale
Director

JRM/EBS/SHH:vog
Attachment

cc: Mr. J. B. Richard (w/a)
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