

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Virgil C. Summer Nuclear Station										DOCKET NUMBER (2) 0 5 0 0 0 3 9 5										PAGE (3) 1 OF 0 2																					
TITLE (4) Inoperability of Residual Heat Removal (RHR) System																																									
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																							
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)								
0 6			1 1			8 5			8 5			0 1			5 0			1 0			7 1			7 8			5			0 5 0 0 0						0 5 0 0 0					
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																							
1		20.402(b)										20.405(e)										50.73(a)(2)(iv)										73.71(b)									
POWER LEVEL (10)		20.405(a)(1)(i)										50.36(e)(1)										50.73(a)(2)(v)										73.71(c)									
1 0 0		20.405(a)(1)(ii)										50.36(e)(2)										50.73(a)(2)(vii)										OTHER (Specify in Abstract below and in Text NRC Form 356A)									
		20.405(a)(1)(iii)										50.73(a)(2)(i)										50.73(a)(2)(viii)(A)																			
		20.405(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(viii)(B)																			
		20.405(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(ix)																			
LICENSEE CONTACT FOR THIS LER (12)																																									
NAME A. R. Koon, Jr., Mgr., Regulatory Compliance																TELEPHONE NUMBER 8 0 3 3 4 5 - 5 2 0 9																									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																									
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																							
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR																			
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)																<input checked="" type="checkbox"/> NO																									

ABSTRACT (Limit to 1400 spaces - i.e., approximately fifteen single-space typewritten lines) (16)

On June 11, 1985, due to personnel error, both trains of the RHR System were declared inoperable concurrently. At 0710 hours, the "A" Train RHR pump was declared inoperable because of the installation of non-seismic scaffolding to support inservice inspection. At approximately 0830 hours, the Shift Supervisor approved performance of surveillance testing of Train "B" Reactor Trip Breaker and Safety Injection (SI) Time Delay (STP-345.076). This test disabled actuation of the Train "B" Load Sequencer, and as such, RHR pump B was inoperable during testing activity.

There were no adverse consequences due to this event. Though the "A" train pump was "Administratively" inoperable, the unit would have started on a valid signal. Also, testing activity of the "B" train could have been terminated and the system returned to operable status if required.

This event was discussed in detail with the responsible Shift Supervisor, and it was reemphasized that the determination of equipment operability must include consideration of seismic qualification.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)
		YEAR SEQUENTIAL NUMBER REVISION NUMBER	
Virgil C. Summer Nuclear Station	0 5 0 0 0 3 9 5 8 5	- 0 1 5 - 0 1	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On June 11, 1985, both trains of the RHR system were declared inoperable concurrently. At 0710 hours, the "A" Train RHR pump was declared inoperable because of the installation of non-seismic scaffolding to support inservice inspection. At approximately 0830 hours, the Shift Supervisor approved performance of surveillance testing of Train "B" Reactor Trip Breaker and Safety Injection (SI) Time Delay (STP-345.076). This test disabled actuation of the Train "B" Load Sequencer, and as such, RHR pump B was inoperable during testing activity.

Testing was performed from 0850 hours to 1047 hours and from 1314 hours to 1418 hours. During the morning test, the technicians encountered a recorder problem, and the system was restored to operable status while the recorder was being repaired. As a result of the aforementioned testing, both RHR trains were inoperable concurrently for a period of 1 hour 57 minutes and 1 hour 4 minutes respectively.

This event was due to a personnel error. The Shift Supervisor was aware that the surveillance test (STP-345.076) was in the extension period as allowed by the Technical Specification. Prior to approval to perform the surveillance test, the Shift Supervisor reviewed the status of the "A" Train and considered the "A" RHR pump to be operable for the purpose of surveillance testing on the "B" Train.

There were no adverse consequences due to this event. Administratively, the "A" Train RHR pump was inoperable; however, the unit would have responded to a valid (SI) signal. In addition, testing of the "B" Train could have been terminated and the system restored to operable status if required.

This event was discussed in detail with the responsible Shift Supervisor, and it was reemphasized that the determination of equipment operability must include consideration of seismic qualification.

In addition, effective June 17, 1985, a "by train" maintenance/testing concept has been established which utilizes the philosophy of scheduling maintenance and testing on only one train during any given week.

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

July 17, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 85-015

Dear Sir:

Attached is a revision to Licensee Event Report #85-015 for the Virgil C. Summer Nuclear Station. This Report is submitted pursuant to the requirements of 10CFR50.73(a)(2)(i).

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

RJB:OSB/lcd
Attachment

cc: V. C. Summer	J. F. Heilman
T. C. Nichols, Jr./O. W. Dixon, Jr.	C. L. Ligon (NSRC)
E. H. Crews, Jr.	K. E. Nodland
E. C. Roberts	R. A. Stough
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