

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Pilgrim Nuclear Power Station DOCKET NUMBER (2) 0 5 0 0 0 2 9 3 PAGE (3) 1 OF 0 2

TITLE (4) Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)							
0	6	1	5	8	5	8	5	0	7	0	8	5	0	5	0	0	0	0

OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																									
N	0 1 0	<table border="1"><tr><td>20.402(b)</td><td>20.406(e)</td><td><input checked="" type="checkbox"/></td><td>80.73(a)(2)(iv)</td><td>73.71(b)</td></tr><tr><td>20.406(a)(1)(i)</td><td>80.38(c)(1)</td><td></td><td>80.73(a)(2)(v)</td><td>73.71(e)</td></tr><tr><td>20.406(a)(1)(ii)</td><td>80.38(c)(2)</td><td></td><td>80.73(a)(2)(vii)</td><td>OTHER (Specify in Abstract below and in Text, NRC Form 306A)</td></tr><tr><td>20.406(a)(1)(iii)</td><td>80.73(a)(2)(i)</td><td></td><td>80.73(a)(2)(viii)(A)</td><td></td></tr><tr><td>20.406(a)(1)(iv)</td><td>80.73(a)(2)(ii)</td><td></td><td>80.73(a)(2)(viii)(B)</td><td></td></tr><tr><td>20.406(a)(1)(v)</td><td>80.73(a)(2)(iii)</td><td></td><td>80.73(a)(2)(ix)</td><td></td></tr></table>												20.402(b)	20.406(e)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)	20.406(a)(1)(i)	80.38(c)(1)		80.73(a)(2)(v)	73.71(e)	20.406(a)(1)(ii)	80.38(c)(2)		80.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)	20.406(a)(1)(iii)	80.73(a)(2)(i)		80.73(a)(2)(viii)(A)		20.406(a)(1)(iv)	80.73(a)(2)(ii)		80.73(a)(2)(viii)(B)		20.406(a)(1)(v)	80.73(a)(2)(iii)		80.73(a)(2)(ix)	
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20.406(a)(1)(ii)	80.38(c)(2)		80.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)																																							
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20.406(a)(1)(v)	80.73(a)(2)(iii)		80.73(a)(2)(ix)																																								

LICENSEE CONTACT FOR THIS LER (12)  
NAME Paul J. Hamilton - Senior Plant Engineer TELEPHONE NUMBER 6 1 7 7 4 6 - 7 9 0 0  
AREA CODE

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	

SUPPLEMENTAL REPORT EXPECTED (14)  
☐ YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NO  
EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 6/15/85, at approximately 0435 hours, an unplanned reactor scram occurred from less than 10% reactor power. The scram was the result of an MSIV isolation on high reactor water level with reactor pressure greater than 600 psig.

Cause of the high level, which was being manually controlled, is personnel error. Corrective action was to counsel personnel.

The unit was returned to service on 6/16/85 at approximately 2109 hours.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Pilgrim Nuclear Power Station Unit #1	DOCKET NUMBER (2) 0 1 5 1 0 0 0 2 1 9 3 8 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		85	014	010	012	OF	012

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 6/15/85, at approximately 0435 hours, an unplanned reactor scram occurred from less than 10% reactor power. The scram was the result of an MSIV isolation with reactor pressure greater than 600 psig. The isolation was initiated by high reactor water level which was being controlled manually when the scram occurred. At the time of the event, the turbine was off-line in preparation for planned maintenance, reactor pressure (which was being decreased) was approximately 700 psig, and the mode switch was in the startup position.

Cause of the event is utility licensed operator error. The error occurred when a turbine bypass valve was cracked open and the water level was slightly high. The expected level "swell" associated with opening the bypass valve combined with fluctuating water level resulted in the high water level MSIV isolation. During and previous to the event, water level was fluctuating as it normally does when at reduced steaming rates. Corrective action was to verbally counsel Operations personnel with regard to the event.

The high water level isolation setpoint is  $\leq 48$ ". The high level alarm was set at 32" and was in and acknowledged. A post-scram review of recorders indicates that the isolation occurred at the expected setpoint.

There were no equipment failures associated with this event and the scram sequence was normal. On 6/16/85, at 2109 hours, the generator was synchronized to the grid after completion of the planned turbine maintenance.

A search of records indicates no previous LER's of a similar nature have been submitted.

This event did not impact the health and safety of the public.

BOSTON EDISON COMPANY  
800 BOYLSTON STREET  
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON  
SENIOR VICE PRESIDENT  
NUCLEAR

July 8, 1985  
BECO Ltr. #85-124

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Docket Number 50-293  
License DPR-35

Dear Sir:

The attached Licensee Event Report 85-014-00, "Reactor Scram," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,

*W D Harrington*  
W. D. Harrington

PH:caw

Enclosure: LER 85-014-00

cc: Dr. Thomas E. Murley  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
531 Park Avenue  
King of Prussia, PA 19406

Standard BECO LER Distribution

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