

WOLF CREEK GENERATING STATION

MONTHLY OPERATING REPORT

MONTH: April YEAR: 1985

Docket No.: STN 50-482

Facility Operating License No.: NPF-32

Report No. 2

Submitted by:

Kansas Gas and Electric Company

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The following report highlights the operating experience of Wolf Creek Generating Station for the month of April, 1985. This report is being provided pursuant to Technical Specification 6.9.1.8.

I. Operating Experience

During the month of April, cold rod drop testing was completed. The plant has been heated up and pressurized to normal operating temperature and pressure, and has entered Mode 3.

Several Engineered Safeguards Features actuations occurred during April. A portion of these were due to personnel errors or spurious signals. Two actuations occurred in response to valid plant conditions generated as a result of personnel actions.

II. Major Safety Related Maintenance Activities

The major safety related maintenance performed during the month of April includes the rework of the Power Operated Relief Valves to increase the clearances between the plugs and the cages and the replacement of the inboard oil seal of a centrifugal charging pump. Some work was performed on the Emergency Diesel Generators including the performance of web deflection tests and inspection of the pillow block bearings.

III. CHANGES, TESTS, AND EXPERIMENTS

The following is a brief description of safety evaluations performed pursuant to 10 CFR 50.59 on changes, tests, and experiments during the month of April.

1. Plant Modification Request (PMR) #00588 - Rework of Ruskin fire dampers due to failure of the dampers to close if flow is present in the ductwork. No unreviewed safety or environmental questions are generated as a result of this change.
2. Plant Modification Request #00637 - Addition of a closure to a fire penetration through a seismic block wall in the Auxiliary Building. No unreviewed safety or environmental questions are generated as a result of this change.
3. Plant Modification Request #00681 - Rework of a leaking check valve, AE-V-121, in the Main Feedwater System. The PMR was evaluated, and it was determined that no unreviewed safety or environmental questions are generated as a result of this change.
4. Plant Modification Request #00606 - Upgrade of fusing protection at containment electrical penetrations. The PMR was evaluated and it was determined that no unreviewed safety or environmental questions are generated by this change provided the modification was completed prior to Mode 4 operation.

5. Plant Modification Request #00743 - Change the storage location of the reactor vessel O-rings to the top of the pressurizer enclosure and change the storage location of the reactor cavity seal ring lifting fixture to the bottom of the fuel pool. No unreviewed safety or environmental questions are generated as a result of this change.
6. Plant Modification Request #00808 - Modify the Power Operated Relief Valves, BB-PCV 455A and 456A, to increase the clearances between the plugs and cages of the valves. The PMR was evaluated, and it was determined that no unreviewed safety or environmental questions are generated as a result of this change, provided the modification was completed prior to Mode 3 operation.
7. Plant Modification Request #00823 - Change cable termination points to support reactor trip signal P-4 interlock logic. The PMR was evaluated and it was determined that no unreviewed safety or environmental questions are generated as a result of this change, provided the modification is completed prior to Mode 2 operation.
8. Plant Modification Request #00816 - Addition of seal table drains to prevent drainage onto the thimble guide tubing. No unreviewed safety or environmental questions are generated as a result of this change.
9. Plant Modification Request #00798 - Replacement of instrument air/backup nitrogen system brass tee with a type 316 stainless tee on a valve in the Auxiliary Feedwater System. No unreviewed safety or environmental questions are generated as a result of this change.
10. Plant Modification Request #00654 - Seal floor penetrations in Spent Resin Sluice Pump Room of the Radwaste Building to prevent possible spread of contamination in accordance with ALARA program. No unreviewed safety or environmental questions are generated as a result of this change.
11. Plant Modification Request #00646 - Relocation of radioactive waste compactor from the Containment Building to the Radwaste Building. No unreviewed safety or environmental questions are generated as a result of this change.
12. Temporary Modification #85-204-EC - Installation of flush adaptors on fuel pool cooling valves EC-V037, EC-V043, and EC-V104 to allow draining of demineralized water from the spent fuel pool. This temporary modification was reviewed and it was determined that no unreviewed safety or environmental questions are generated as a result of this change, provided the flush adaptors are removed prior to the first refueling outage.
13. Temporary Procedure Change Form #MA 85-798 to SU8-BB13, Rev. 0 - "Reactor Vessel Level Indication System". - Written to allow for the collection of additional data with different combinations of Reactor Coolant Pumps running. No unreviewed safety or environmental questions are generated as a result of this change.

14. Temporary Procedure Change form #MA 85-894 to SU8-BB13, Rev. 0 - "Pressurizer Relief Valve Test" - Written to allow test instrumentation change from digital thermometer to a surface pyrometer. This test instrument will be used for trending purposes only. No unreviewed safety or environmental questions are generated as a result of this change.
15. Temporary Procedure Change Form #MA 85-878 to SU8-BB13, Rev. 0, "Pressurizer Relief Valve Test" - Written to allow testing to be performed at a lower temperature. The lower Reactor Coolant System temperature will not affect the test results. No unreviewed safety or environmental questions are generated as a result of this change.

points to support reactor trip criteria. See Appendix A for details.

OPERATING DATA REPORT

DOCKET NO. STN 50/482
WOLF CREEK GENERATING STATION
KANSAS GAS AND ELECTRIC COMPANY
DATE 5-01-85
COMPLETED BY M. Williams
TELEPHONE 316-364-8831

OPERATING STATUS

1. Reporting Period: April, 1985 Gross Hours in Reporting Period: 719
2. Currently Authorized Power Level (MWt): 170 Max. Depend. Capacity (MWe-Net): 0
Design Electrical Rating (MWe-Net): 1186
3. Power Level to Which Restricted (If Any) (Mwe-Net): 0
4. Reasons for restriction (If Any): 5% License Issued

	This Month	Yr to Date	Cumulative
5. Number of Hours Reactor was Critical	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
6. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. Hours Generator on Line	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
8. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
10. Gross Electrical Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
11. Net Electrical Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
12. Reactor Service Factor	<u>N/A</u>	<u></u>	<u></u>
13. Reactor Availability Factor	<u>N/A</u>	<u></u>	<u></u>
14. Unit Service Factor	<u>N/A</u>	<u></u>	<u></u>
15. Unit Availability Factor	<u>N/A</u>	<u></u>	<u></u>
16. Unit Capacity Factor (Using MDC)	<u>N/A</u>	<u></u>	<u></u>
17. Unit Capacity Factor (Using Design MWe)	<u>N/A</u>	<u></u>	<u></u>
18. Unit Forced Outage Rate	<u>N/A</u>	<u></u>	<u></u>

19. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of each): None
20. If Shut Down at End of Report Period, Estimated Date of Startup: N/A
21. Units in test Status (Prior to Commercial Operation):

Forecast	Achieved
Initial Criticality	<u>5-22-85</u>
Initial Electricity	<u>6-11-85</u>
Commercial Operation	<u>8-10-85</u>

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WOLF CREEK GENERATING STATION
KANSAS GAS AND ELECTRIC COMPANY
DATE 5-01-85
COMPLETED BY M. Williams
TELEPHONE 316-364-8831

MONTH April, 1985

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)		DAY AVERAGE DAILY POWER LEVEL (MWe-Net)	
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	<u>0</u>
3	<u>0</u>	19	<u>0</u>
4	<u>0</u>	20	<u>0</u>
5	<u>0</u>	21	<u>0</u>
6	<u>0</u>	22	<u>0</u>
7	<u>0</u>	23	<u>0</u>
8	<u>0</u>	24	<u>0</u>
9	<u>0</u>	25	<u>0</u>
10	<u>0</u>	26	<u>0</u>
11	<u>0</u>	27	<u>0</u>
12	<u>0</u>	28	<u>0</u>
13	<u>0</u>	29	<u>0</u>
14	<u>0</u>	30	<u>0</u>
15	<u>0</u>	31	<u>N/A</u>
16	<u>0</u>		

INSTRUCTIONS:

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnotes to explain the apparent anomaly.

UNIT SHUTDOWN AND POWER REDUCTIONS

DOCKET NO. STN 50/482

WOLF CREEK GENERATING STATION

KANSAS GAS AND ELECTRIC COMPANY

DATE 5-01-85

COMPLETED BY M. Williams

TELEPHONE 316-364-8831

REPORT MONTH April, 1985

[illegible]

SUMMARY:

Fuel Load and 5% power license issued 3-11-85. Fuel loading has been completed. Startup testing is in progress.

(1) REASON:

- A: EQUIPMENT FAILURE (EXPLAIN)
B: MAINTENANCE OR TEST
C: REFUELING
D: REGULATORY RESTRICTION
E: OPERATOR TRAINING AND LICENSE EXAMINATION
F: ADMINISTRATIVE
G: OPERATIONAL ERROR (EXPLAIN)
H: OTHER (EXPLAIN)

(2) METHOD:

1. MANUAL
2. MANUAL SCRAM
3. AUTOMATIC SCRAM
4. OTHER (EXPLAIN)

KANSAS GAS AND ELECTRIC COMPANY

WOLF CREEK GENERATING STATION

UNIT NO. 1

MONTH April

SUMMARY OF OPERATING EXPERIENCE

Listed below in chronological sequence is a summary of operating experiences for this month which required load reduction or resulted in significant non-load related incidents.

<u>DATE</u>	<u>TIME</u>	<u>EVENT</u>
April 1, 1985	1445	Received engineering evaluation on pressurizer heatup rate. Exited Technical Specification Action Statement 3.4.9.2.
April 6, 1985	2400	Plant commencing preparations for full flow rod drop testing.
April 7, 1985	1038	Received Control Room Ventilation Isolation Signal (CRVIS) along with Fuel Building Ventilation Isolation Signal (FBVIS) due to personnel error. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
	1133	Reset from CRVIS and FBVIS.
April 8, 1985	0849	Received CRVIS due to radiation monitor not being placed in bypass prior to commencement of surveillance activity. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
	0923	Reset from CRVIS.
April 9, 1985	1409	Received Containment Purge Isolation Signal (CPIS) and CRVIS due to a mechanical problem. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
	1503	Reset from CPIS and CRVIS.
April 12, 1985	1114	Received CPIS, FBVIS, and CRVIS. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
	1245	Determined CPIS, FBVIS and CRVIS was due to 120v distribution panel feeder breaker ground fault.

<u>DATE</u>	<u>TIME</u>	<u>EVENT</u>
	1855	Reset from FBVIS and CRVIS.
	2215	Received CRVIS due to spurious trip of a chlorine monitor. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
	2227	Reset from CRVIS.
April 17, 1985	0753	Plant entered Mode 4.
April 19, 1985	1619	Received inadvertent Safety Injection Signal (SIS) due to personnel error during surveillance testing. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
	1811	Reset from SIS in accordance with approved plant procedures.
April 21, 1985	1922	Received CRVIS due to a chlorine monitor running out of paper. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
	2008	Reset from CRVIS.
April 26, 1985	1854	Received Auxiliary Feed Actuation Signal (AFAS) due to a personnel error. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
	1953	Reset from AFAS.
	2239	Plant entered Mode 3.
April 28, 1985	1550	Received SIS due to a valid steamline low pressure signal resulting from a personnel error. Reportable per 10 CFR 50.72 and 10 CFR 50.73. Declared an Unusual Event and termination of an Unusual Event in accordance with Emergency Plan Implementing Procedures (EPP's).
April 29, 1985	0153	Received Main Feedwater Isolation Signal due to a valid High Level Signal in Steam Generator "C", resulting from a personnel error. Reportable per 10 CFR 50.72 and 10 CFR 50.73.
April 30, 1985	1148	Received SIS due to low steamline pressure. The signal was the result of hand held radio transmissions. Reportable per 10 CFR 50.72 and 10 CFR 50.73. Declared an Unusual Event and termination of an Unusual Event in accordance with Emergency Plan Implementing Procedures (EPP's).



KANSAS GAS AND ELECTRIC COMPANY

GLENN L KOESTER
VICE PRESIDENT - NUCLEAR

May 13, 1985

COPY FOR

Director, Office of Resource Management
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-111

Re: Docket No. STN 50-482

Subj: April, 1985 Monthly Operating Report

Gentlemen:

Enclosed is the April, 1985 Monthly Operating Report for Wolf Creek Generating Station. This submittal is being made in accordance with the requirements of Technical Specification 6.9.1.8.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

GLK:bb

Enc.

xc:PO'Connor (2), w/a

JTaylor (12), w/a

JCummins, w/a

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