

LICENSEE EVENT REPORT (LER)

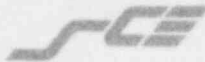
FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3										DOCKET NUMBER (2) 0 5 0 0 0 3 6 2				PAGE (3) 1 OF 0 1			
TITLE (4) FUEL HANDLING ISOLATION SYSTEM (FHIS) ACTUATIONS																	
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)				
0 8	2 2	8 5	8 5	0 2 3	0 0	0 9	2 3	8 5					0 5 0 0 0 0				
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)															
1		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)			
POWER LEVEL (10)		0 5 5				20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)		73.71(c)	
		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)							
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)							
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)							
LICENSEE CONTACT FOR THIS LER (12)																	
NAME H. E. MORGAN, STATION MANAGER										TELEPHONE NUMBER AREA CODE 7 1 4 4 9 2 - 7 7 0 0							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																	
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC							
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO					

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 8/22/85, at 1145 with no fuel in the Spent Fuel Pool (SFP) (EIIS Code DB), filling of SFP was initiated. Fuel Handling Isolation System (FHIS)(EIIS Code VG) Train A and Train B actuated at 1158 and 1205, respectively. The actuations were verified to be valid by confirming that Airborne Monitors 3RT-7822 and 3RT-7823 (EIIS Component Code MON) readings were above their 130 cpm above background alarm setpoint, and SFP fill operation was stopped. The Fuel Handling Building post-accident cleanup filter system automatically actuated as designed, and the airborne activity was removed. Both FHIS trains were reset at 1235, and fill of the SFP was re-initiated. FHIS Trains A and B actuated again at 1250 and 1255, respectively, and filling of the SFP was secured. Since the FHIS is not required when there is no fuel in the SFP, at 1435 both FHIS monitors were placed in alarm defeat to prevent unnecessary actuations until filling of the SFP was completed.

The increase in background radiation in the Fuel Handling Building was caused by a small amount of gas in the Chemical Volume and Control (CVCS) piping which became entrained in the blended water flow from the CVCS to the SFP. The source of the gas was minor back-leakage from the Volume Control Tank (VCT). Unit 3 VCT gas has significantly greater activity than Unit 2 VCT due to fuel leakage. Although Unit 2 has a similar CVCS piping configuration, no FHIS actuations have occurred during SFP fill operations. Unit 3 was removed from service on September 14 for refueling. During this outage, fuel inspection and reconstitution will be performed to minimize fuel leakage for the next fuel cycle.

Since all FHIS components functioned as designed, there are no safety consequences with this event.



Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P. O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

H. E. MORGAN
STATION MANAGER

September 23, 1985

TELEPHONE
(714) 368-6241

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-362
30-Day Report
Licensee Event Report No. 85-023
San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for four occurrences involving actuations of the Fuel Handling Isolation System (FHIS). Since these events involved the same components, system, cause, and method of discovery, these events have been combined into a single report in accordance with NUREG-1022. Neither the health and safety of plant personnel nor the health and safety of the public was affected by this event.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER No. 85-023

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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