

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 4 8 3				PAGE (3) 1 OF 0 2	
TITLE (4) Lo Lo Steam Generator Level Reactor Trip															
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)		
0 6	2 0	8 5	8 5	0 3	1	0 0	0 7	1 7	8 5					0 5 0 0 0	
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)													
1		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)			
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)		73.71(c)			
1 0 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)															
NAME Charles D. Naslund - Superintendent, I&C										TELEPHONE NUMBER AREA CODE 3 1 1 4 6 1 7 6 1 - 1 8 1 5 1 0 1 0					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 6/20/85 at 0811 CDT a lo-lo level in Steam Generator (S/G) 'C' initiated a Reactor Trip, Auxiliary Feedwater Actuation Signal (AFAS) and S/G Blowdown Isolation Signal (SGBDIS) as designed. The plant was in Mode 1 (Power Operation) at 100% power at the time of the event.

Technicians were performing an Analog Channel Operational Test (ACOT) on pressure detector loop P535 of S/G 'C.' Procedure requires that Main Steam Outlet Flow Channel AB-FI-532A be selected as the controlling channel while AB-FI-533A is inoperable for the ACOT. However, this step was omitted due to a personnel error and AB-FI-533A was left in service. A low steam flow signal was sent to Main Feedwater Regulating Valve AE-FCV-530 causing it to close. Manual control of AE-FCV-530 was taken in an attempt to restore S/G level. However, the lo-lo level in S/G 'C' occurred shortly thereafter resulting in the Reactor Trip, AFAS and SGBDIS. Operators recovered from the trip and by 0822 the plant was stable in Mode 3, Hot Standby.

S/G ACOT procedures now verify that selector switches are positioned properly by use of a sign-off step and a "Test in Progress" tag.

There was no damage to plant equipment or release of radioactivity as a result of this incident. In no way was the public health or safety affected.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Callaway Plant Unit 1	0 5 0 0 0 4 8 3	8 5	- 0 3 1	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 6/20/85 at approximately 0811 CDT a lo-lo level in Steam Generator (S/G) 'C' initiated a Reactor Trip, Auxiliary Feedwater Actuation Signal (AFAS) and S/G Blowdown Isolation Signal (SGBDIS) as designed. The plant was in Mode 1 (Power Operation) at 100% power at the time of the event.

I&C technicians were performing an Analog Channel Operational Test (ACOT) on one of the three pressure detector loops of S/G 'C' (loop P535) which provide a low steam pressure safety injection signal and steam line isolation signal. During this test the procedure requires that flow selector AB-FS/532C (IEEE Std. 805-1983 System SB; IEEE Std. 803A-1983 Component - FS) be placed to the F532 position. This places Main Steam Outlet Flow Channel AB-FI-532A (IEEE Std. 805-1983 System - SB; IEEE Std. 803A-1983 Component - FI) in service while Main Steam Outlet Flow Channel AB-FI-533A (IEEE Std. 805-1983 System - SB; IEEE Std. 803A-1983 Component - FI) is inoperable for the ACOT on loop P535. However, this step of the procedure was not performed due to a personnel error and AB-FI-533A was left in service. A simulated low steam flow signal was sent to Main Feedwater Regulating Valve AE-FCV-530 (IEEE Std. 805-1983 System - SJ; IEEE Std. 803A-1983 Component - FCV) causing it to close. This resulted in a steam flow/feed flow mismatch alarm and S/G level deviation alarm on S/G 'C.' When these alarms were received the balance of plant operator took manual control of AE-FCV-530 in an attempt to restore S/G level. The lo-lo level in S/G 'C' occurred shortly thereafter resulting in the Reactor Trip, AFAS and SGBDIS.

Following the Reactor Trip operators performed emergency operating procedures E-0, Reactor Trip or Safety Injection, and ES-0.1, Reactor Trip Recovery. By 0822 on 6/20/85 the plant was stable in Mode 3, Hot Standby, commencing reactor startup operations.

To prevent recurrence, S/G steam line pressure ACOT procedure revisions have been made which now verify that selector switches are positioned properly by use of a sign-off step and a "Test in Progress" tag on the selector switches.

The Reactor Trip, AFAS and SGBDIS responded as designed when the lo-lo level signal on S/G 'C' was received. There was no damage to plant equipment or release of radioactivity as a result of this incident. In no way was the public health or safety affected.

Previous occurrences: none

UNION ELECTRIC COMPANY
CALLAWAY PLANT

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July 17, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-1142

Gentlemen:

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 85-031-00
LO LO STEAM GENERATOR LEVEL REACTOR TRIP

The enclosed Licensee Event Report is submitted pursuant to
10 CFR 50.73(a)(2)(iv) concerning an inadvertent Reactor Trip
initiated during steam generator surveillance activities.

S. E. Miltenberger
S. E. Miltenberger
Manager, Callaway Plant

MB SEM
CDN/WRR/RCW/drs
Enclosure

cc: Distribution attached

IE22
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cc distribution for ULNRC-1142

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N. Date