

December 9, 1996

Mr. H. G. Stanley
Site Vice President
Braidwood Nuclear Power Station
Commonwealth Edison Company
R.R. #1, Box 84
Braceville, IL 60407

SUBJECT: NRC INSPECTION REPORT NO. 50-456/96017(DRS)

Dear Mr. Stanley:

This refers to the inspection conducted by Mr. M. Holmberg on October 21-25, 1996, November 5, 1996 and November 26, 1996. The inspection included a review of activities pertaining to the examination of steam generator tubes for your Braidwood Unit 1 facility during a mid-cycle outage. At the conclusion of the inspection, the findings were discussed with those members of your staff identified in the enclosed report.

The indications identified during eddy current examination of the Unit 1 steam generator tubes were predominantly single circumferentially oriented indications located in the tube roll transition region. These indications are indicative of outside diameter stress corrosion cracking. Your efforts to evaluate tube integrity by assessing growth rates for this type of cracks by historical eddy current data reviews/comparisons and in-situ pressure testing was indicative of an aggressive program. However, the inspector had concerns with eddy current indications detected this outage which were present but not identified during prior outages.

In accordance with 10 CFR 2.790 of the Commission's regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC Public Document Room.

We will gladly discuss any questions you have concerning these inspections.

Sincerely,

Geoffrey E. Grant, Director
Division of Reactor Safety

Docket No. 50-456

Enclosure: Inspection Report
No. 50-456/96017(DRS)

See Attached Distribution

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NAME	Holmberg/nph	<i>per</i>	Lanksbury <i>DDT</i>		Kropp <i>for MTP</i>		Grant <i>BC for</i>	
DATE	11/7/96		11/2/96		<i>11/1/96</i>	<i>12/5/96</i>	<i>12/9/96</i>	

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