

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | 1

(PLEASE PRINT OR TYPE ALL REQ

INFORMATION)

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

V A S P S 2 0 0 - 0 0 0 0 0 - 0 0 4 1 1 1 1 4 5

LICENSEE CODE LICENSE NUMBER LICENSE TYPE JO CAT 58

REPORT SOURCE L 6 0 5 0 0 0 2 8 1 7 0 4 1 3 8 3 8 0 5 1 3 9 3 9

DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

With Unit 2 at 47% power, steam flow channel, FI-2474, was indicating less than the redundant channel. This is contrary to T.S.-3.7.B, Table 3.7.2 and is reportable per T.S.-6.6.2.b.(2). The redundant channel remained operable and FI-2474 was placed in trip, thereby establishing the required degree of redundancy. The health and safety of the public were not affected.

SYSTEM CODE H B 11 CAUSE CODE E 12 CAUSE SUBCODE B 13 COMPONENT CODE I N S T R U 14 COMP. SUBCODE T 15 VALVE SUBCODE Z 16

EVENT YEAR 8 3 SEQUENTIAL REPORT NO. 0 2 1 OCCURRENCE CODE 0 3 REPORT TYPE L REVISION NO. 0

LER/RO REPORT NUMBER 17 ACTION TAKEN A 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 ATTACHMENT SUBMITTED Y 23 NFRD-4 FORM SUB. Y 24 PRIME COMP. SUPPLIER A 25 COMPONENT MANUFACTURER R 3 7 0 25

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

A steam leak was founded in one of the transmitter root valves, hence FI-2474 was sensing a lower differential pressure. The defective root valve was replaced.

FACILITY STATUS 1 POWER 0 4 7 29 OTHER STATUS 30 N/A METHOD OF DISCOVERY 31 A Operator Observation

AMOUNT OF ACTIVITY 25 N/A LOCATION OF RELEASE 28 N/A

PERSONNEL EXPOSURES NUMBER 0 0 0 32 TYPE Z 33 DESCRIPTION 29 N/A

ACTIVITIES DESCRIPTION 31 N/A

DEGREE OF DAMAGE TO FACILITY 34 TYPE N/A

FACILITY 35

36

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PDR ADOCK 05000281
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USE ONLY

(-04) 57-134

ATTACHMENT 1
SURRY POWER STATION, UNIT NO. 2
DOCKET NO: 50-281
REPORT NO: 83-021/03L-0
EVENT DATE: 04-13-83

TITLE OF THE EVENT: STEAM FLOW CHANNEL 2474 INOPERABLE

1. Description of the Event

With the unit at 47% power after a startup following a reactor trip, it was observed that steam flow indicator FI-2-474 was reading low and thereby declared inoperable. This event is contrary to T.S.-3.7.B Table 3.7-2 and is reportable per T.S.-6.6.2.b.(2).

2. Probable Consequences and Status of Redundant Equipment

The steam line flow instrumentation provided signals to actuate the Safety Injection System upon sensing the effects of a steam line break. SIS actuation following a steam line break is designed to occur upon sensing high steam line flow in coincidence with low reactor coolant average temperature or low steam line pressure. Since the bi-stable for the failed instrument (2-FI-474) was placed in the trip mode and the redundant channel remained operable, the health and safety of the public would not have been affected.

3. Cause

A steam leak was discovered in the side of one of the transmitter root valves. This caused a smaller differential pressure resulting in a low indicated flow. When unit power approached 100%, the error created by the steam leak was within acceptable limits.

4. Immediate Corrective Action

The bistable test switches for the affected channel were placed into test. The transmitter was calibrated, however, a discrepancy still existed between the two steam flow channels.

5. Subsequent Corrective Action

The defective root valve was discovered and replaced during the outage that began 4-16-83.

6. Action Taken to Prevent Recurrence

None deemed necessary.

7. Overall Implications

None.