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the southern electric system

NED-85-664
2113N

September 17, 1985

Director of Nuclear Reactor Regulation
Attention: Mr. John F. Stolz, Chief
Operating Reactors Branch No. 4
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

NRC DOCKET 50-321
OPERATING LICENSE DPR-57
EDWIN I. HATCH NUCLEAR PLANT UNIT 1
FINAL DESIGN REPORT - RECIRCULATION AND
RHR SYSTEMS ANALYSES AND REPAIRS

Gentlemen:

By letter dated December 10, 1984, Georgia Power Company submitted information regarding the analyses and repairs of Recirculation (Recirc) System and Residual Heat Removal (RHR) System piping welds which were inspected during the 1984 Hatch Unit 1 maintenance/refueling outage and found to have unacceptable flaw indications. Included as Enclosure 2 to that submittal was Revision A of NUTECH report no. XGP-09-106, "Design Report For Evaluation And Disposition Of IGSCC Flaws At Plant E. I. Hatch Unit 1." The subject NUTECH report was provided in preliminary draft form as discussed in the aforementioned letter.

Since our submittal of December 10, 1984, NUTECH has issued the final design report. Enclosed herein are ten (10) copies of Revision 0 of NUTECH report no. XGP-09-106. The enclosed NUTECH report supercedes the preliminary draft version submitted as Enclosure 2 to our December 10, 1984 letter. Some of the major differences between Revision 0 of the NUTECH design report and the preliminary draft version include:

- o Weld overlay as-built dimensions (Table 2.1).
- o Weld overlay configuration drawings provided for new and existing overlays (Figures 2.1 through 2.7).
- o Code stress results (Tables 5.1 though 5.5).

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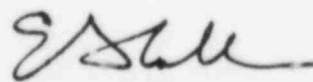
- o Weld stress information for 1984 analyses (Table 5.6).
- o Weld overlay induced shrinkage stresses (Table 5.7).
- o Low toughness material concerns discussion.

The overall conclusions of the adequacy of the repairs, etc. reached in your letters dated January 3, 1985 and February 14, 1985 relative to the inspections of Recirc and RHR piping and resultant corrective actions taken during the 1984 Hatch Unit 1 maintenance/refueling outage are not changed as a result of the enclosed final NUTECH design report. Submittal of the enclosed final design report should complete all necessary submittals to NRC relative to piping inspections and repairs during the subject outage.

By copy of this letter, a copy of the final NUTECH design report is being supplied to NRC Region II for their review.

Should you have any questions in this regard, please contact this office.

Sincerely yours,


for L. T. Gucwa

JAE/mb

Enclosures (10)

xc: (All w/l encl.)

J. T. Beckham, Jr.
H. C. Nix, Jr.
J. N. Grace (NRC Region II)
Senior Resident Inspector