

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Point Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 3 6 1 OF 0 2										PAGE (3) 1 OF 0 2	
TITLE (4) Loss of Low Temperature Overpressure Protection																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)							
0 6	1	5	8 5	8 5	0 1 0	0 0	0 7	1 5	8 5						0 5 0 0 0						
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																			
5		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)							
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				<input checked="" type="checkbox"/> 50.73(a)(2)(v)				73.71(c)							
0 0 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
		20.405(a)(1)(iii)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(A)											
		20.405(a)(1)(iv)				50.73(a)(2)(iii)				50.73(a)(2)(viii)(B)											
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME Steve Stadnick, Plant Engineer, X4427										TELEPHONE NUMBER											
										AREA CODE											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs											
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a Containment Building Integrated Leak Rate Test (ILRT) at the end of the 1985 refueling outage the Low Temperature Over Pressure (LTOP) protection control circuit was found out of service. This was caused by two (2) pressurizer pressure transmitters that were isolated and vented for the ILRT. These transmitters feed a signal to the Reactor Protection System (RPS) which would initiate an opening signal to the two Power Operated Relief Valves (PORVs) if pressure had exceeded 440 PSIA.

The immediate corrective action was to manually initiate opening of the PORVs by pulling the RPS High Pressurizer Pressure Modules. Additional corrective action has been an ILRT procedure revision to open the PORVs prior to venting the pressure transmitters.

The above incident is a violation of Technical Specification Action Statement 3.4.9.3. This statement requires two operable PORVs or a RCS vent of at least 1.3 square inches. The RCS was vented for the ILRT, however vent size was 1.1 square inches and the time of the venting and pressure transmitters is not established, (occurred on same day). Based on this, a lack of LTOP protection could have occurred for a period of time.

8507290207 850715
PDR ADOCK 05000336
S PDR

JE22
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 3/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Millstone Point Unit 2

0 5 0 0 0 3 3 6

8 5

-

0

1

0

-

0

0

0

2

OF

0

2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

At the end of the 1985 refueling outage a Containment Building Integrated Leak Rate Test (ILRT) was being performed. As a prerequisite for the ILRT, Instrument and Control personnel isolate and vent various pressure and level transmitters in containment. Pressure transmitters PT 103 and PT 103-1 (Pressurizer Pressure) were isolated and vented on 6/13/85. Operations performed a valve line-up on the Reactor Coolant System (RCS) for the ILRT on the same day, 6/13/85. This particular line-up included opening three (3) vent valves on the RCS. The combined area of these vents is 1.1 square inches.

On 6/15/85 at 1130 during the pressurization phase of the ILRT, Operations realized that the pressure transmitters were isolated for the test and no Low Temperature Over Pressure (LTOP) Protection existed. This is required whenever the temperature of one or more of the RCS cold legs is less than or equal to 275°F per Technical Specification 3.4.9.3. The limiting condition for operation requires two Power Operated Relief Valves (PORVs) with a lift setting less than or equal to 440 PSIA or a RCS vent greater than or equal to 1.3 square inches.

Based on the above, insufficient LTOP protection existed for up to 36 hours.

The cause of this incident is a procedure inadequacy. The procedure instructs personnel to isolate and vent instrumentation, but makes no provision for the consequences of isolating the pressure transmitters.

Operations personnel took immediate corrective action by manually pulling two (2) of the the four (4) RPS Channel High Pressurizer Pressure Bistable modules from their location in the cabinet. This action opened both PORVs and established LTOP protection. As an action to prevent recurrence the ILRT procedure has been revised to make provision for LTOP protection prior to isolation of the pressure transmitters.

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Selden Street, Berlin, Connecticut

P.O. BOX 270
HARTFORD, CONNECTICUT 06141-0270
(203) 666-6911

July 15, 1985
MP-7092

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. DPR-65
Docket No. 50-336
Reportable Occurrence RO 50-336/85-010

Gentlemen:

This letter forwards the Licensee Event Report 85-010 required to be submitted within thirty (30) days pursuant to paragraphs 50.73 (a)(2) (iv) and 50.73 (a)(2)(v) for a condition that resulted in a violation of Technical Specification requirements.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

A handwritten signature in cursive script, appearing to read 'Wayne D. Romberg'.

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/SLS:mo

Attachment: LER RO 50-336/85-010

cc: Dr. T. E. Murley, Region I

IER22
1/1