



Nebraska Public Power District

COOPER NUCLEAR STATION
P.O. BOX 96, BROWNVILLE, NEBRASKA 68321
TELEPHONE (402)825-3811
FAX (402)825-5205

NLS960238

December 12, 1996

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555-0001

Dear Sir:

Cooper Nuclear Station Licensee Event Report 96-009 Supplement 1 is forwarded as an attachment to this letter.

Sincerely,

M. F. Peckham
Plant Manager

/wrv

Attachment

cc: Regional Administrator
USNRC - Region IV

Senior Project Manager
USNRC - NRR Project Directorate IV-1

Senior Resident Inspector
USNRC- Cooper Nuclear Station

NPG Distribution

INPO Records Center

W. Turnbull
MidAmerica Energy

TEW
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NRC FORM 366 <small>(4-95)</small>		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 04/30/98 <small>ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (IT-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.</small>	
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)					
FACILITY NAME (1) COOPER NUCLEAR STATION			DOCKET NUMBER (2) 05000298		PAGE (3) 1 OF 1
TITLE (4) Appendix R Safe Shutdown Analysis Vulnerabilities					
EVENT DATE (5) MONTH DAY YEAR 07 25 96		LER NUMBER (6) YEAR SEQUENTIAL NUMBER REVISION NUMBER 96 -- 009 -- 01		REPORT DATE (7) MONTH DAY YEAR 12 12 96	
OTHER FACILITIES INVOLVED (8)					
		FACILITY NAME		DOCKET NUMBER	
		FACILITY NAME		DOCKET NUMBER	
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)			
		20.2201(b)		20.2203(a)(2)(v)	
		20.2203(a)(1)		20.2203(a)(3)(i)	
POWER LEVEL (10) 100		20.2203(a)(2)(i)		20.2203(a)(3)(ii)	
		20.2203(a)(2)(ii)		20.2203(a)(4)	
		20.2203(a)(2)(iii)		50.36(c)(1)	
		20.2203(a)(2)(iv)		50.36(c)(2)	
				<input checked="" type="checkbox"/> 50.73(a)(2)(vii)	
				<input type="checkbox"/> 50.73(a)(2)(viii) <input type="checkbox"/> 50.73(a)(2)(ix) <input type="checkbox"/> 50.73(a)(2)(x) <input type="checkbox"/> 73.71 <input checked="" type="checkbox"/> OTHER <small>Specify in Abstract below or in NRC Form 366A</small>	
LICENSEE CONTACT FOR THIS LER (12)					
NAME William R. Victor, Licensing and Compliance Specialist				TELEPHONE NUMBER (Include Area Code) (402) 825-3811	
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
SUPPLEMENTAL REPORT EXPECTED (14)					
<input checked="" type="checkbox"/> YES <small>(If yes, complete EXPECTED SUBMISSION DATE).</small>				<input type="checkbox"/> NO	
				EXPECTED SUBMISSION DATE (15)	
				MONTH DAY YEAR 01 24 97	
ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)					
<p>As part of a corrective action for LER 94-016 Supplement 1, a reevaluation of the Cooper Nuclear Station (CNS) Appendix R Safe and Alternate Shutdown Analysis Report has been in progress. It was recently determined that certain fire induced vulnerabilities did not have acceptable coping strategies. Specifically, postulating a fire on 931' Reactor Building North and assuming multiple ground cable faults, the Reactor Core Isolation Cooling (RCIC) System controller could become disabled. CNS Procedure 5.4.3.1 credits RCIC for safe shutdown following a fire in this area. During this postulated event, other systems would not be available to compensate for the loss of RCIC control. In addition, assuming multiple ground cable faults during an alternate shutdown fire could result in the failure of the Diesel Generator No. 2 (DG-2) control power fuse prior to isolation of the DG-2 control circuits from the fire area. Mitigation procedures did not direct operators to replace the DG-2 control power fuse, nor do NRC regulations allow crediting this action (not shutdown repair) without prior NRC review and approval. Compensatory measures have been established that have provided a commensurate level of safety in coping with a plant fire event. On July 25, 1996, an ENS notification was provided pursuant to 10 CFR 50.72(b)(2)(iii)(A). In a supplemental ENS notification made on August 13, 1996, CNS reported additional findings from the Appendix R reevaluation that four types of assumptions may not have been fully addressed in the original Appendix R Safe and Alternate Shutdown Analysis Report. These previously identified vulnerabilities have been corrected. An additional ENS supplement was made on November 12, 1996, which discussed the potential inability of RHR-MOV-MO39B to open from its Alternate Shutdown Panel due to an interlock with RHR-MOV-MO15B during an alternate shutdown fire. This would challenge the ability to initiate Suppression Pool Cooling within the credited 3-hour time period, and required entry into Technical Specification 3.2.1.2. CNS plans to modify the RHR-MOV-MO39B control circuitry to eliminate this vulnerability.</p> <p>The CNS Appendix R reevaluation effort is continuing. This LER is being submitted in abstract form pending the completion of this reanalysis, and complies with the NRC notification requirement of CNS Technical Specification 3.2.1.2. Additional supplements will be provided, as necessary, based on the ongoing Appendix R reevaluation.</p>					

Correspondence No: NLS960238

The following table identifies those actions committed to by the District in this document. Any other actions discussed in the submittal represent intended or planned actions by the District. They are described to the NRC for the NRC's information and are not regulatory commitments. Please notify the Licensing Manager at Cooper Nuclear Station of any questions regarding this document or any associated regulatory commitments.

COMMITMENT	COMMITTED DATE OR OUTAGE
Additional [LER] supplements will be provided, as necessary, based on the ongoing Appendix R reevaluation [a final non-abstract supplement will be provided following the completion of the CNS Appendix R Safe and Alternate Shutdown reanalysis].	None