

LICENSEE EVENT REPORT (LER)

APPROVED COMB NO 2180-010W
EXPIRES - 6/31/81

ACTIVITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
INDIAN POINT, UNIT 2	0 5 0 0 0 1 2 4 17	1 OF 013

TITLE (4) Late Surveillance Test

EVENT DATE (8)			LER NUMBER (8)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME		DOCKET NUMBER(S)							
0	8	2	1	8	5	8	5	-	0	0	8	-	0	5	0	0	0		
0	8	2	1	8	5	8	5	-	0	0	8	-	0	5	0	0	0		

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)					
POWER LEVEL (10)	7.9	20.402(a)		20.406(a)		60.73(a)(2)(iv)	73.71(a)
		20.406(a)(1)(i)		60.36(a)(1)		60.73(a)(2)(v)	73.71(a)
		20.406(a)(1)(ii)		60.36(a)(2)		60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Test NRC Form 364A)
		20.406(a)(1)(iii)	✓	60.73(a)(2)(i)		60.73(a)(2)(vii)(A)	
		20.406(a)(1)(iv)		60.73(a)(2)(ii)		60.73(a)(2)(vii)(B)	
		20.406(a)(1)(v)		60.73(a)(2)(iii)		60.73(a)(2)(iv)	

LICENSER CONTACT FOR THIS LER (12)					
NAME	TELEPHONE NUMBER				
JOHN R. ELLWANGER	<table border="1"> <tr> <td>AREA CODE</td> <td></td> </tr> <tr> <td>9114</td> <td>51261-511812</td> </tr> </table>	AREA CODE		9114	51261-511812
AREA CODE					
9114	51261-511812				

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	
A	A A	F U		Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (16)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)					
<input checked="" type="checkbox"/>	NO				

ABSTRACT (LIMIT IS 1600 CHARACTERS) (S. approximately fifteen single-space typewritten lines) (18)

On August 21, 1985 a rod drop occurred during a control rod insertion test. The resultant Xenon buildup caused a flux tilt which necessitated a power reduction. As a result, a Technical Specification requirement for a daily heat balance could not be performed for the major part of the day and was ultimately inadvertently omitted on August 21, 1985.

Following the transient, reactor power was limited to a maximum of 81%. The high flux trips were set conservatively at approximately 90% for a 9 hour period. A heat balance performed on the following day indicated that the nuclear instrumentation had remained conservatively (approximately 2%) within calibration the previous day.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (3)

PAGE (3)

INDIAN POINT, UNIT 2

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TEXT (if more space is required, use additional NRC Form 366A) (17)

Plant and System Identification:

Westinghouse 4-loop Pressurized Water Reactor - 900 Mwe.

Identification of Occurrence:

Missed Surveillance Test - Heat Balance

Event Date: 8/21/85Reportability Determination Date: 8/26/85Report Due Date: 9/20/85

This report was initiated by SOR 85-357

Description of Occurrence:

Table 4.1-1 of the Technical Specifications requires a daily heat balance to be performed to calibrate the nuclear power range monitors. On August 21, 1985 at 2:02 a.m. during the performance of a control rod exercise test, rod M-4 dropped into the core causing an automatic turbine runback. An LCO condition was entered in that the Quadrant Power Tilt Ratio (QPTR) exceeded permissible Technical Specification limits. At 7:10 a.m. the rod was fully retrieved and re-aligned. At 12:27 p.m. the QPTR decreased to a level permitted by the Technical Specifications. Subsequent Xenon oscillation caused re-entry into an LCO for about 14 hours until 4:33 a.m. on 8/22/85.

At 7:59 a.m. on August 21, 1985, reactor power had decreased to 45%. Reactor power was then slowly increased to 80%. On August 21, 1985 the required heat balance could not be performed because of the required power level changes and Xenon oscillations until the afternoon when the plant had sufficiently stabilized. During the latter part of the day the heat balance was inadvertently omitted due to pre-occupation on the part of the operators with recovery from the transient. It should be noted that, in accordance with the Tech Spec requirement, the power range nuclear instrument system high flux trips were set conservatively at approximately 90% for a 9 hour period on 8/21/85.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/86

FACILITY NAME (1)

INDIAN POINT, UNIT 2

DOCKET NUMBER (2)

0 5 0 0 6 2 4 7

LER NUMBER (5)

YEAR SEQUENTIAL REVISION
NUMBER NUMBER NUMBER

8 5 - 0 1 0 8 - 0 1 0

PAGE (3)

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TEXT (1) more space is required, use additional NRC Form 360A (17)

Apparent Cause of Occurrence

The rod drop and the subsequent Xenon oscillations were such that a heat balance test would have produced meaningless results for a significant part of the time on August 21, 1985, although a retrospective review of the data indicates that it would have been possible to perform a valid test during the latter part of the day. The failure to perform a daily heat balance test was not recognized by the operators in the aftermath of the rod drop and the subsequent Xenon oscillations with which the operators were occupied.

Analysis of Occurrence

A heat balance performed on 8/22/85 indicated that the nuclear instrumentation remained within calibration limits on 8/21/85. Therefore, no safety hazard existed as a result of the missed surveillance.

Corrective Action

The operations staff was reminded of the obligation to operate the plant in accordance with the Technical Specifications regardless of extenuating circumstances.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

September 20, 1985

Re: Indian Point Unit No. 2
Docket No. 50-247
LER-85-008-00

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-85-008-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,

 J. D. O'TOOLE

attach.

cc: Dr. Thomas E. Murley,
Regional Administrator - Region I
U. S. Nuclear Regulatory Commission
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