



SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, Box 15830, Sacramento, California 95813; (916) 452-3211

August 9, 1979

Mr. R. H. Engelken  
U. S. Nuclear Regulatory Commission  
Region V Office of Inspection & Enforcement  
1990 North California Boulevard  
Walnut Creek Plaza, Suite 202  
Walnut Creek, CA 94596



RE: Operating License DPR-54  
Docket No. 50-312  
Reportable Occurrence 79-08

Dear Mr. Engelken:

In accordance with Technical Specifications for Rancho Seco Nuclear Generating Station, Section 6.9.4.2d, and Regulatory Guide 1.16, Revision 4, Section C.2.b.4, the Sacramento Municipal Utility District is hereby submitting a thirty-day report of Reportable Occurrence 79-08.

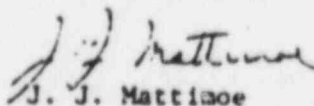
On July 19, 1979, during the performance of SP 203.05G, Monthly Decay Heat Removal Venting Surveillance, leakage was observed from the outboard seal of the B Decay Heat pump. The leakage was found to be in excess of the 0.63 gph allowable per Technical Specifications Section 4.5.3.1. As a result, the B Decay Heat System was determined inoperable and removed from service. The redundant system was tested satisfactory per SP 203.05A, Quarterly Decay Heat Removal Pump P-261A Surveillance and Inservice Test.

The seal was replaced and the system was tested per SP 203.09, Decay Heat Removal Leakage Surveillance, and SP 203.05B, Quarterly Decay Heat Removal Pump P-261B Surveillance and Inservice Test. The results of these tests proved acceptable and the system was declared operable. The total time that the system was out of service was within the 48 hours allowed by Technical Specifications Section 3.3.2 for continued reactor operation.

The pump is a single stage, 3000 gpm centrifugal pump manufactured by Babcock & Wilcox, Canada. The seal is a "John Crane" type 1B 4 1/2 inch shaft seal manufactured by Crane Packing Co., Ltd.

There were no plant transients or power reductions associated with this event.

Respectfully submitted,

  
J. J. Mattiace

Assistant General Manager  
and Chief Engineer

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