



PSE&G Public Service
Electric and Gas
Company

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Robert L. Mittl General Manager
Nuclear Assurance and Regulation

July 19, 1985

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20814

Attention: Mr. Walter Butler, Chief
Licensing Branch 2
Division of Licensing

Gentlemen:

SAFETY EVALUATION REPORT
OPEN AND CONFIRMATORY ITEM STATUS
HOPE CREEK GENERATING STATION
DOCKET NO. 50-354

Attachment 1 is a current list which provides a status of the open and confirmatory items identified in Sections 1.7 and 1.8 of the Safety Evaluation Report (SER). Items identified as "complete" are those for which PSE&G has provided responses and no confirmation of status has been received from the staff. We will consider these items closed unless notified otherwise. In order to permit timely resolution of items identified as "complete" which may not be resolved to the staff's satisfaction, please provide a specific description of the issue which remains to be resolved.

Enclosed for your review and approval (see Attachment 3) are the resolutions to the SER items listed in Attachment 2. This information will be incorporated, as required, into Amendment 12 of the HCGS FSAR.

Should you have any questions or require any additional information on these items, please contact us.

Very truly yours,

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PDR ADOCK 05000354
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Attachments
The Energy People

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Director of Nuclear
Reactor Regulation

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7/19/85

C D. H. Wagner
USNRC Licensing Project Manager (w/attach.)

A. R. Blough
USNRC Senior Resident Inspector (w/attach.)

M P84 154/04 1/2

ATTACHMENT 1

<u>Item No.</u>	<u>Subject</u>	<u>Status</u>	<u>R. L. Mittl to A. Schwencer ltr. dated</u>
OI-1	Riverborne Missiles	Completed	1/31/85, 2/22/85, 5/8/85
OI-2	Equipment Qualification	Partial Response	2/1/85, 2/20/85, 2/28/85, 3/1/85, 6/11/85, 6/6/85, 6/25/85 (2), 6/28/85, 7/16/85
OI-3	Preservice Inspection Program	Partial Response	2/14/85 & 3/19/85
OI-4	GDC 51 Compliance	Completed	3/12/85
OI-5	Solid-State Logic Modules	Partial Response	6/5/85
OI-6	Postaccident Monitoring Instrumentation	Completed	5/14/85
OI-7	Minimum Separation Between Non-Class IE Conduit and Class IE Cable Trays	Partial Response	4/4/85
OI-8	Control of Heavy Loads	Closed	1/18/85 (SSER 1)
OI-9	Alternate and Safe Shutdown	NRC Action	
OI-10	Delivery of Diesel Generator Fuel Oil and Lube Oil	Closed	Amendment 8 (SSER 1)
OI-11	Filling of Key Management Positions	Open	
OI-12	Training Program Items		
	(a) Initial Training Program	Completed	1/7/85
	(b) Regualification Training Program	Completed	12/28/84, 4/26/85 (Revised Program)
	(c) Replacement Training Program	Completed	1/7/85
	(d) TMI Issues I.A.2.1, I.A.3.1, and II.B.4	Completed	1/7/85
	(e) Nonlicensed Training Program	Completed	1/7/85
OI-13	Emergency Dose Assessment Computer Model	Completed	1/7/85

<u>Item No.</u>	<u>Subject</u>	<u>Status</u>	<u>R. L. Mittl to A. Schwencer ltr. dated</u>
OI-14	Procedures Generation Package	Partial Response	1/28/85 & 4/10/85
OI-15	Human Factors Engineering	Partial Response	4/10/85
C-1	Feedwater Isolation Check Valve Analysis	Open	
C-2	Plant-unique Analysis Report	Completed	1/8/85, 1/31/85 6/12/85 & 7/2/85
C-3	Inservice Testing of Pumps and Valves	Completed	7/12/85
C-4	Fuel Assembly Accelerations	Completed	Amendment 8
C-5	Fuel Assembly Liftoff	Completed	Amendment 8
C-6	Review of Stress Report	Completed	7/19/85
C-7	Use of Code Cases	Completed	12/17/84
C-8	Reactor Vessel Studs and Fasteners	Completed	5/24/85 Rev. 1
C-9	Containment Depressurization Analysis	NRC Review	
C-10	Reactor Pressure Vessel Shield Annulus Analysis	NRC Review	
C-11	Drywell Head Region Pressure Response Analysis	NRC Review	
C-12	Drywell-to-Wetwell Vacuum Breaker Loads	NRC Review	
C-13	Short-Term Feedwater System Analysis	Complete	4/22/85
C-14	Loss-of-Coolant-Accident Analysis	Completed	3/1/85 & 6/12/85
C-15	Balance-of-Plant Testability Analysis	Completed	Amendment 8
C-16	Instrumentation Setpoints	Partial Response	2/15/85
C-17	Isolation Devices	Open	

R. L. Mittl to
A. Schwencer
ltr. dated

<u>Item No.</u>	<u>Subject</u>	<u>Status</u>	
C-18	Regulatory Guide 1.75	NRC Review	
C-19	Reactor Mode Switch	NRC Review	
C-20	Engineered Safety Features Reset Controls	Partial Response	7/19/85
C-21	High Pressure Coolant Injection Initiation	Completed	7/1/85
C-22	IE Bulletin 79-27	Completed	Amendment 8
C-23	Bypassed and Inoperable Status Indication	NRC Review	
C-24	Logic for Low Pressure Coolant Injection Interlock Circuitry	Open	
C-25	End-of-Cycle Recirculation Pump Trip	Completed	3/1/85
C-26	Multiple Control System Failures	NRC Review	
C-27	Relief Function of Safety/Relief Valves	Completed	2/15/85
C-28	Main Steam Tunnel Flooding Analysis	Completed	5/24/85
C-29	Cable Tray Separation Testing	Completed	4/4/85
C-30	Use of Inverter as Isolation Device	Completed	3/7/85
C-31	Core Damage Estimate Procedure	Completed	6/24/85
C-32	Continuous Airborne Particulate Monitors	Completed	7/1/85
C-33	Qualifications of Senior Radiation Protection Engineer	Open	
C-34	Onsite Instrument Information	Completed	7/1/85
C-35	Airborne Iodine Concentration Instruments	Completed	6/18/85

<u>Item No.</u>	<u>Subject</u>	<u>Status</u>	R. L. Mittl to A. Schwencer <u>ltr. dated</u>
C-36	Emergency Plan Items	Partial Response	11/9/84, 1/16/85, 2/7/85, 4/4/85 & 7/19/85
C-37	TMI Item II.K.3.18	Partial Response	3/1/85 & 4/22/85

ATTACHMENT 2

<u>ITEM NO.</u>	<u>SER SECTION</u>	<u>SUBJECT</u>
C-6	5.2.1.1	Review of Stress Report
C-20	7.3.2.6	Engineered safety features reset controls.

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ATTACHMENT 3

M P85 27/10 6-mr

SER CONFIRMATORY ISSUE 6 (5.2.1.1)

Review of stress report

However, the staff requires the applicant to comply with Article NA-3260 of the 1971 Edition of Section III of the ASME Code. This is a confirmatory item.

RESPONSE

Article NA-3260 of the 1971 Edition of Section III of the ASME Code provides guidance on the review of stress reports. A review was performed for all RCPB components and it has been determined that they are in compliance with the requirements of Article NA-3260.

SER CONFIRMATORY ISSUE 20 (7.3.2.6)

ENGINEERED SAFETY FEATURES RESET CONTROLS

The staff requires that the applicant submit the revised electrical schematic/elementary diagrams for all safety-related equipment (both NSSS and BOP) that has been modified to prevent its reverting back to its normal (non-emergency) mode after an ESF reset. The staff will confirm that the valves noted above remain in their emergency mode upon an ESF reset, therefore, conforming with the recommendation of IE Bulletin 80-06. Additionally, the staff will require preoperational tests to demonstrate that all equipment remains in its emergency mode upon removal of the actuating signal and/or resetting of the various isolating or actuation signals. This is a confirmatory item.

RESPONSE

FSAR Sections 14.2.12.1.45 and 14.2.12.1.47 have been revised to verify that all equipment remains in its emergency mode upon removal of the actuating signal and/or resetting of the various isolating or actuation signals. The revised electrical schematic/elementary diagrams will be provided upon completion of the required modifications.

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2. All logic combinations initiating automatic closure are verified.
3. All auxiliary actions, including alarms, computer alarms, fan starts, and damper action are checked.

d. Acceptance Criteria

1. All containment isolation valves shall close automatically upon receipt of their isolation signals.
2. Valve closure times shall be within the requirements of Table 6.2-16.
3. *on removal of actuating signal (ESF) and/or resetting of the isolation or actuation signal, equipment remains in emergency mode.*
- 4 3. All alarms, annunciators, instruments, and interlocks shall function in accordance with the system electrical schematics.

14.2.12.1.46 SP-Process Radiation Monitoring

a. Objective

The test objective is to verify the capability of the process radiation monitoring system to detect radioactivity in the monitored process lines.

b. Prerequisites

1. All permanently installed equipment, relays, and instrumentation have been functionally operated and calibrated.
2. Sections of the monitored systems required for the test are operational.
3. Ac and dc power as available.

- (c) The dc system loads respond and operate properly throughout the test period.

d. Acceptance Criteria

1. On a loss of standby bus power, diesel generators start and load shed and sequence as specified in Table 8.3-1.
2. On total loss of offsite power, diesel generators start, shed loads, and accept the sequenced loads as specified in Table 8.3-1.
3. Integrated ECCS response demonstrates the ability of RHR and core spray to inject water into the reactor vessel at the flow rates and within the times specified in the GE Preoperational test specification.
4. Integrated ECCS response in conjunction with simulated LOCA/LOSP signals demonstrates the ability of the diesel generators to maintain ECCS load while they provide rated flow to the vessel within the time specified in the GE preoperational test specification.
5. Dc system loads operate at voltage levels which occur during integrated ECCS response to a simulated LOCA event in conjunction with LOSP and inoperable battery chargers.
6. *On removal of actuating signal (ESF) and/or resetting of the isolation or actuation signal equipment remains in emergency mode.*

14.2.12.1.48 Fuel Handling and Vessel Servicing Equipment

a. Objectives

1. To verify all interlocks and logic associated with the refueling platform and service platform.
2. To verify use of tools for servicing control rods, fuel assemblies, incore detectors.