

APPENDIX B

U. S. NUCLEAR REGULATORY COMMISSION
REGION IV

NRC Inspection Report: 50-458/85-42

CP: CPPR-145

Docket: 50-458

Licensee: Gulf States Utilities
P. O. Box 2951
Beaumont, Texas 77704

Facility Name: River Bend Station

Inspection At: River Bend Site, St. Francisville, Louisiana

Inspection Conducted: June 3-7, 1985

Inspectors: W R Bennett 7/17/85
W. R. Bennett, Reactor Inspector Date
Project Section A, Reactor Project Branch 1

W R Bennett for 7/17/85
C. C. Harbuck, Reactor Inspector Date
Project Section A, Reactor Project Branch 1

Approved: J. P. Jaudon 7/17/85
J. P. Jaudon, Chief, Project Section A, Date
Reactor Project Branch 1

Inspection Summary

Inspection Conducted June 3-7, 1985 (Report 50-458/85-42)

Areas Inspected: Routine unannounced inspection of preoperational test witnessing and preoperational test results evaluation. The inspection involved 56 inspector-hours onsite by two NRC inspectors.

Results: Within the two areas inspected, one violation was identified (Failure to tag lifted leads, paragraph 2).

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DETAILS1. Persons ContactedGulf States Utilities (GSU)

*D. L. Andrews, Director of Training
 *T. P. Anthony, Senior Mechanical Engineer
 *C. L. Ballard, Projects Supervisor
 *J. Bleau, Radwaste Specialist
 *J. F. Booker, Manager - Engineering, Fuels, and Licensing
 *W. J. Cahill, Senior Vice President
 *L. Campo, Radwaste Specialist
 *T. L. Crouse, Manager-Quality Assurance (QA)
 *P. J. Dautel, Licensing Staff Assistant
 *J. C. Deddens, Vice President
 D. R. Derbonne, Preoperations Testing Supervisor
 *R. Easlick, Radwaste Supervisor
 *L. A. England, Supervisor - Licensing
 *P. E. Freehill, Superintendent Startup and Test (SU&T)
 *D. R. Gipson, Assistant Plant Manager
 *K. C. Hodges, System QA Supervisor
 D. Jernigan, SU&T Engineer
 *G. R. Kimmell, Supervisor Operations QA
 *G. V. King, Plant Services Supervisor
 *M. Lett, Radiation Protection/Radwaste
 T. Pao, SU&T Engineer
 *T. F. Plunkett, Plant Manager
 *S. R. Radebaugh, Assistant Superintendent SU&T
 *D. M. Reynerson, Executive Assistant
 *C. Rohrmann, Technical Training Coordinator
 *J. E. Spivey, QA Engineer
 *R. B. Stafford, Director Quality Services
 K. E. Suhrke, Manager Project Planning and Coordination
 *P. F. Tomlinson, Director Operations QA

Other Contractor Contacts

K. Bullen, General Electric SU&T Engineer
 *F. W. Finger, Stone & Webster (SWEC) Project Manager, Preliminary Test Organization
 *B. R. Hall, SWEC Assistant Superintendent Field Quality Control

The NRC inspectors also contacted other site personnel including administrative, clerical, operations, testing, and inspection personnel.

*Denotes those attending the exit interview on June 7, 1985.

2. Preoperational Test Witnessing

The objective of this portion of the inspection was to verify that testing is conducted in accordance with approved procedures. The NRC inspectors observed performance of portions of 1-PT-508, "Reactor Protection System Preoperational Test," and 1-PT-200, "Remote Shutdown System Preoperational Test." The testing was performed and documented in accordance with approved procedures.

During observation of 1-PT-508, the NRC inspector found four leads in panel 821 which were lifted without having temporary alteration tags as required by River Bend Procedure TI-17, "Test Controls" Revision 8, January 18, 1985. Further investigation determined that the leads had been lifted for several days. This is an apparent violation (458/8542-01). In addition to the aforementioned leads, several other leads were not landed and the NRC inspector could not determine whether they were lifted leads or spare leads. Discussions with several licensee personnel in the area did not disclose the purpose of the leads. Subsequent investigation by the licensee determined that all suspect leads were spares. The licensee determined that a means of identifying spare leads was needed and committed to the NRC inspector to develop and implement a means of spare lead identification. This is considered an open item (458/8542-02).

No other violations or deviations were identified in this portion of the inspection.

3. Preoperational Test Results Evaluation

The objective of this portion of the inspection was to assure that the licensee is performing an adequate evaluation of test results and to assure that all test data is either within acceptance criteria or that deviations are properly dispositioned. The NRC inspector reviewed completed preoperational test procedure 1-PT-210, "Integrated Emergency Core Cooling System and Loss of Offsite Power." The NRC inspector discovered several minor documentation problems which were corrected by the licensee. One test exception was outstanding and was properly documented. All test exceptions reviewed by the NRC inspector were properly dispositioned and documented.

No violations or deviations were identified in this portion of the inspection.

4. Exit Interview

An exit interview was conducted on June 7, 1985, with those personnel denoted in paragraph 1 of this report. At this exit interview the NRC inspectors summarized the scope and findings of this inspection.