

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Wolf Creek Generating Station

DOCKET NUMBER (2)

0 5 0 0 0 4 8 2 1 OF 0 1 2

TITLE (4)

Reactor Protection/ESF Actuation - Reactor Trip and Feedwater Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)				
0	5	0	1	8	5	8	5	0	2	3	0	5	0	0	0
0	5	0	1	8	5	8	5	0	2	3	0	5	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10)	01010	20.402(b)	20.406(e)	X	80.73(a)(2)(iv)	73.71(b)					
		20.403(a)(1)(i)	80.36(a)(1)		80.73(a)(2)(v)	73.71(c)					
		20.405(a)(1)(ii)	80.36(a)(2)		80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)					
		20.405(a)(1)(iii)	80.73(a)(2)(i)		80.73(a)(2)(vii)(A)						
		20.405(a)(1)(iv)	80.73(a)(2)(ii)		80.73(a)(2)(vii)(B)						
		20.405(a)(1)(v)	80.73(a)(2)(iii)		80.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)
NAME
Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services
TELEPHONE NUMBER
AREA CODE
3116 31641-181831

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) ☐ NO ☒
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0533 CDT on May 1, 1985, a Reactor Protection System (RPS) trip signal and an Engineered Safety Features Actuation Signal (ESFAS) were initiated when test equipment used to support a startup test procedure was being removed from a process control cabinet. This resulted in an Over-temperature Delta-T (OTdT) Reactor trip, which in turn resulted in a Feedwater Isolation Signal (FWIS). All control rods were fully inserted at the time of this event, and all required engineered safety features equipment responded properly.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of this event. Reactor Coolant System temperature and pressure was 545 degrees F and approximately 1900 psig respectively.

This event posed no threat to the public health and safety.

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PDR ADOCK 05000482
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Wolf Creek Generating Station	0 5 0 0 0 4 8 2 8 5	—	0 2 3	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 0533 CDT on May 1, 1985, a Reactor Protection System (RPS) trip signal and an Engineered Safety Features Actuation Signal (ESFAS) were initiated due to a series of events involving a startup test procedure and a surveillance test procedure.

Surveillance procedure STS-IC-502A, "Calibration of Pressurizer Pressure Transmitters", had previously removed pressurizer pressure transmitter BB-PT455 from service. As expected, this resulted in an Over-temperature Delta-T (OTdT) Reactor trip signal for channel one (pre-trip), but not an actual Reactor Trip since 2-out-of-4 coincidence is required.

Concurrent with this evolution, was the removal of resistance temperature detector (RTD) test boxes from the process control cabinets following the completion of startup test procedure SU7-BB06.1, "RTD/TC Cross Calibration Test". This evolution likewise results in an OTdT Reactor Trip signal, one channel at a time, as each associated channel RTD test box is removed. At 0533, two OTdT channels were simultaneously actuated, one due to the out-of-service pressure transmitter, and one due to the removal of an RTD test box, causing an OTdT Reactor trip. The resultant opening of the reactor trip breakers satisfied a protection system permissive (P-4), which in conjunction with low Reactor Coolant System average temperature (Tavg), initiated a Feedwater Isolation Signal (FWIS). All required engineered safety features equipment responded properly.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of this event, all control rods were fully inserted, plant Tavg was 545 degrees F, and Reactor Coolant System pressure was approximately 1900 psig.

The actuated systems were restored to normal per plant procedures at 0555.

Investigation revealed that the individual test procedures involved in this event contained adequate precautions and sufficient detail necessary to prevent such an occurrence, when performed individually. This cognitive personnel error occurred however, due to having multiple test activities, involving overlapping instrumentation, in progress simultaneously. Consequently, all plant testing was temporarily suspended. During this time, the need to more strictly control plant testing activities and the necessity to remain continually aware of the specific impact which these test activities will have on plant operations was stressed to all operating and test personnel. In addition, the importance of more thoroughly briefing operating personnel on specific test evolutions was emphasized to personnel involved in various test activities.

This event posed no threat to the public health and safety.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

May 30, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-133
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-023-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

Glenn L. Koester
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

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