

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station										DOCKET NUMBER (2) 0 5 0 0 0 4 8 2 1 OF 0 3										PAGE (3) 1				
TITLE (4) Engineered Safety Features Actuation - Safety Injection and Main Steamline Isolation																								
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)										
0	4	3	0	8	5	8	5	0	2	2	0	0	0	5	3	0	8	5	0 5 0 0 0					
OPERATING MODE (9) 3			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																					
POWER LEVEL (10) 0 0 0		20.402(b)		20.405(a)(1)(i)		20.405(a)(2)(iv)		20.405(a)(2)(v)		20.405(a)(2)(vi)		20.405(a)(2)(vii)(A)		20.405(a)(2)(vii)(B)		20.405(a)(2)(viii)		73.71(b)		73.71(c)		OTHER (Specify in Abstract below and in Text, NRC Form 305A)		
		20.406(a)(1)(ii)		20.406(a)(1)(iii)		20.406(a)(1)(iv)		20.406(a)(1)(v)		20.406(a)(2)(i)		20.406(a)(2)(ii)		20.406(a)(2)(iii)		20.406(a)(2)(iv)								
		20.406(a)(1)(vi)		20.406(a)(1)(vii)		20.406(a)(1)(viii)		20.406(a)(1)(ix)		20.406(a)(2)(v)		20.406(a)(2)(vi)		20.406(a)(2)(vii)		20.406(a)(2)(viii)								
LICENSEE CONTACT FOR THIS LER (12)																								
NAME Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services												TELEPHONE NUMBER AREA CODE 3 1 6 3 6 4 - 8 8 3 1												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)												
YES (If yes, complete EXPECTED SUBMISSION DATE)												MONTH DAY YEAR												
X NO																								
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																								

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (18)

On April 30, 1985, at 1143 CDT, a Safety Injection and Main Steamline Isolation occurred due to a low steamline pressure signal generated by interference from a hand-held radio transmission. The plant was in Mode 3, Hot Standby, prior to initial criticality with the Reactor Coolant System (RCS) at normal operating temperature, 557 degrees F, and pressure, 2250 psig, at the time of the event.

All required Engineered Safety Features equipment responded properly. An Unusual Event was declared and subsequently cancelled. The appropriate federal, state, and local agencies were notified. The Safety Injection was terminated per plant procedures by 1155 CDT.

The use of radio communications within the plant has been severely restricted due to this incident.

There were no radiation levels present in excess of normal background, and at no time did conditions develop which posed a threat to the health and safety of the public.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0500048285-022-0002 OF 03	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On April 30, 1985, at 1143 CDT, an Engineered Safety Features (ESF) actuation occurred, resulting in a Safety Injection (SI) into the Reactor Coolant System (RCS), and a Main Steamline Isolation. The initiating signal was low steamline pressure. At the time of this event, the plant was in Mode 3, Hot Standby, prior to initial criticality. The RCS was at the normal operating temperature, 557 degrees F, and pressure, 2250 psig. Pressurizer level was being manually maintained at approximately 22 percent and pressure was being maintained automatically. The secondary side was being maintained in steady state conditions by dumping steam to the condenser.

When the Safety Injection Signal occurred, all ESF equipment required to actuate with the plant in Mode 3 responded properly. The pressurizer pressure control system limited the pressure transient in the RCS to a maximum of 2328 psig, and the pressurizer level increased to 38 percent. The minimum RCS temperature during this event was 551 degrees F. The pressurizer Power Operated Relief Valves did not lift during this transient.

The operators carried out the appropriate steps of procedure EMG E-0, "Safety Injection", and when steady state conditions were reached the appropriate steps of the SI recovery procedure ES-03, "SI Termination", were performed. The Safety Injection was terminated by approximately 1155 CDT.

An Unusual Event was declared and terminated. The appropriate federal, state, and local agencies were notified in accordance with the Emergency Plan Implementing Procedures.

The Post Trip Review revealed that steamline pressure never reached the trip setpoint for low steamline pressure, although the indications on steam generators "B" and "C" did show a momentary decrease in pressure from 1070.2 and 1065.8 psig to 746.5 and 925.8 psig respectively. Shortly after this pressure dip, the pressure returned to normal. Although the decrease in steam pressure did not reach the low pressure setpoint, the rate sensitive nature of the signal circuit caused an anticipatory trip. Thus, the initiating signal of low steamline pressure was due to the step decrease in steam pressure.

Subsequent investigations into this event confirmed that the step decrease in the steam pressure signal was caused by interference from an operator's hand held radio transmission. An operator was standing in the area of the steamline pressure transmitters and had keyed his radio just prior to the initiation of this event. This scenario has been reenacted with the result being an identical response. At the time of reenactment, the plant was below the P-11 interlock point, and no actuation of ESF equipment occurred, but the required logic for a low steamline pressure signal was induced.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

As a result of this event, a review of in-plant communications has been initiated. The use of radios within the plant has been severely restricted and is now more stringently controlled.

There were no radiation levels present in excess of normal background, and at no time during this event did conditions develop that would pose a threat to the health and safety of the public.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

May 30, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-132

Re: Docket No. STN 50-482

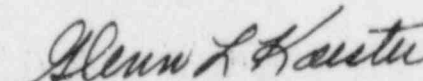
Subj: Licensee Event Report 85-022-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,


Glenn L. Koester
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

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