

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Brunswick Steam Electric Plant Unit 1	0 5 0 0 0 3 2 5 1	OF 0 1

TITLE (4)

Primary Containment Group 8 Isolation of Reactor Shutdown Cooling

EVENT DATE (6)			LER NUMBER (8)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)												
0	5	02	8	5	8	5	0	2	3	0	0							0	5	0	0	0				

OPERATING MODE (8)		5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following): (11)				
POWER LEVEL (10)		0 0 0	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
			20.405(a)(1)(i)	50.36(c)(1)		50.73(a)(2)(v)	73.71(c)
			20.406(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
			20.406(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)	
			20.406(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	
			20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER	
M. J. Pastva, Jr., Regulatory Compliance Technician	AREA CODE	
	9 1 9	4 5 7 - 2 3 1 1 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (18)

During a Unit 1 refueling/maintenance outage on 05-02-85, at 1538, the unit Control Operator discovered that a primary containment Group 8 isolation of the Residual Heat Removal (RHR) System's shutdown cooling inboard primary containment isolation valve, 1-E11-F009, had occurred. The unit reactor cavity was flooded, the fuel pool gates were removed, and the Fuel Pool Cooling System and Reactor Water Cleanup System were available for reactor decay heat removal. RHR flow recorder traces showed the isolation was sealed in for approximately 15 to 20 minutes before discovery of the event. Within five minutes of the event discovery, the isolation was reset and RHR shutdown cooling was reestablished following determination that an actual isolation condition did not exist.

This event is attributed to a spurious interruption of the logic circuitry to the reactor steam dome pressure instrument, 1-B32-PS-N018A-1. An investigation was inconclusive in determining whether ongoing work activities in the vicinity of the associated Control Room RHR isolation actuation relays resulted in the event. The instrument actuation setpoint of N018A-1 was also checked and found within required tolerances.

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Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

May 31, 1985

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SERIAL: BSEP/85-0965

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-85-023

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/clh

Enclosure

cc: Dr. J. N. Grace

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