

50-282/306



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20585-0001

October 28, 1996

Mr. Roger O. Anderson, Director
Licensing and Management Issues
Northern States Power Company
414 Nicollet Mall
Minneapolis, MN 55401

SUBJECT: CLOSEOUT FOR NORTHERN STATES POWER COMPANY'S RESPONSE TO GENERIC LETTER 92-01, REVISION 1, SUPPLEMENT 1 FOR THE PRAIRIE ISLAND UNITS 1 AND 2 NUCLEAR GENERATING PLANTS (TAC NOS. M92719 AND M92720)

Dear Mr. Anderson:

On May 19, 1995, the NRC issued Generic Letter 92-01, Revision 1, Supplement 1 (GL 92-01, Rev. 1, Supp. 1), "Reactor Vessel Structural Integrity." In GL 92-01, Rev. 1, Supp. 1, the NRC requested that nuclear licensees perform a review of their reactor pressure vessel structural integrity assessments in order "to identify, collect, and report any new data pertinent to [the] analysis of structural integrity of their reactor pressure vessels (RPVs) and to assess the impact of that data on their RPV integrity analyses relative to the requirements of Section 50.60 of Title 10 of the *Code of Federal Regulations* (10 CFR 50.60), 10 CFR 50.61, Appendices G and H to 10 CFR Part 50 (which encompass pressurized thermal shock (PTS) and upper shelf energy (USE) evaluations), and any potential impact on low temperature overpressure (LTOP) limits or pressure-temperature (P-T) limits."

More specifically, in GL 92-01, Rev. 1, Supp. 1, the NRC requested that addressees provide the following information in their responses:

- (1) a description of those actions taken or planned to locate all data relevant to the determination of RPV integrity, or an explanation of why the existing data base is considered complete as previously submitted;
- (2) an assessment of any change in best-estimate chemistry based on consideration of all relevant data;
- (3) a determination of the need for the use of the ratio procedure in accordance with the established Position 2.1 of Regulatory Guide (RG) 1.99, Revision 2, for those licensees that use surveillance data to provide a basis for the RPV integrity evaluation; and
- (4) a written report providing any newly acquired data as specified above and (1) the results of any necessary revisions to the evaluations of RPV integrity in accordance with the requirements of 10 CFR 50.60, 10 CFR 50.61, Appendices G and H to 10 CFR Part 50, and any potential impact on the LTOP and P-T limits in the technical specifications, or
(2) a certification that previously submitted evaluations remain valid.

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Revised evaluations and certifications were to include consideration of Position 2.1 of RG 1.99, Revision 2, as applicable, and any new data. The information in Reporting Item (1) was to be submitted within 90 days of the issuance of the GL. The information in Reporting Items (2) — (4) was to be submitted within 6 months of the issuance of the GL.

The NRC staff has noted that NSP submitted the information requested in Reporting Item (1) on August 17, 1995, and requested in Reporting Items (2) — (4) on November 17, 1995. Since NSP has submitted the requested information and has indicated that the previously submitted evaluations remain valid, the staff considers the RPV integrity data for the Prairie Island Units 1 and 2 Nuclear Generating Plants to be complete at this time. The staff therefore concludes that no additional information regarding the structural integrity of the RPV at the Prairie Island Units 1 and 2 Nuclear Generating Plants is available at this time, and that your efforts regarding GL 92-01, Rev. 1, Supp. 1 are complete. This completes all actions related to the referenced TAC Number.

Thank you for your cooperation.

Sincerely,



Beth A. Wetzel, Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

cc: See next page

Mr. Roger O. Anderson, Director
Northern States Power Company

Prairie Island Nuclear Generating
Plant

cc:

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Sincerely,

Original signed by
Beth A. Wetzel, Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

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cc: See next page

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