

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Wolf Creek Generating StationDOCKET NUMBER (2)
0 5 0 0 0 4 8 2 1 OF 0 2TITLE (4)
Feedwater Isolation and Auxiliary Feedwater Actuation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)												
0	6	0	9	8	5	8	5	0	4	3	0	0	0	7	0	9	8	5	0	5	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
2		20.402(b)		20.405(c)	X	50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)		50.38(c)(1)		50.73(a)(2)(v)		73.71(c)			
POWER LEVEL (10)	01 01 2	20.405(a)(1)(ii)		50.38(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)
NAME
Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services
TELEPHONE NUMBER
AREA CODE
3 1 6 3 6 4 1 8 1 8 3 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B	SI	J F I C I V I	C I 6 I 3 I 5	N					

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) ☐ NO ☒
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 9, 1985, at approximately 0106 CDT, a Feedwater Isolation Signal, Main Turbine trip and Steam Generator Feed Pump Turbine Trip Signals occurred due to hi-hi water level in Steam Generator "A". An Auxiliary Feedwater Actuation Signal also occurred due to the trip of both Main Feedwater Pumps.

At the time of this event, the plant was in Mode 2 (Startup) at approximately two (2) percent Reactor power. When Main Feedwater Pump "A" was placed in service at approximately 0100 CDT, Steam Generator water levels began rapidly increasing. The Main Feedwater Control Valve Bypass valves which were being used to supply feedwater to the Steam Generators were manually closed. However, water levels continued to increase to the hi-hi level trip setpoint on Steam Generator "A" due to leakage through the closed Main Feedwater Control valves.

All required Engineered Safety Features equipment responded properly, and normal feedwater flow was restored at 0120 CDT.

There was no damage to plant equipment or release of radioactivity as a result of this event and at no time did conditions develop that could have posed a threat to the health or safety of the public.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0 5 0 0 0 4 8 2 8 5 - 0 4 3 - 0 0 0 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
					OF	

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On June 9, 1985, at approximately 0106 CDT, a Feedwater Isolation Signal, Main Turbine trip and Steam Generator Feed Pump (SGFP) Turbine Trip Signals were initiated by a hi-hi water level signal in Steam Generator (S/G) [AB-SG] "A". Due to the SGFP Turbine Trip Signals, an Auxiliary Feedwater Actuation (AFAS) was also initiated.

Prior to this event, the plant was in Mode 2, Startup, at approximately two (2) percent Reactor power. Main Feedwater Pump [SJ-P] "A" had been placed in service at approximately 0100 CDT, and Steam Generator water levels began to rapidly increase. Feedwater flow to the Steam Generators was via the Main Feedwater Control Valve Bypass valves [SJ-FCV] which were in manual control. These valves were promptly closed to limit the water level increase. However, S/G levels continued to increase until the setpoint for hi-hi level in S/G "A" was reached at approximately 0106 CDT. This initiated a Feedwater Isolation, Main Turbine trip, SGFP Turbine Trips and an Auxiliary Feedwater Actuation. The Main Turbine [TA-TRB] and Main Feedwater Pump "B" were not in operation at the time.

All required Engineered Safety Features equipment responded properly.

S/G "A" water level reached a maximum of approximately eighty-three (83) percent. Main Feedwater Pump "A" was restarted at approximately 0120 CDT and the actuated systems were returned to normal in accordance with operating procedures by approximately 0125 CDT.

The cause of this event was leakage through the closed Main Feedwater Control Valves [SJ-FCV]. This leakage, coupled with the controlling procedure requirement of maintaining S/G levels at fifty-five (55) to sixty-five (65) percent in preparation for a power level increase, permitted S/G "A" water level to reach the hi-hi level setpoint of seventy-eight (78) percent before actions to isolate the Main Feedwater Control valves could be completed.

The Main Feedwater Control Valves were manufactured by Copes-Vulcan, Inc. Maintenance actions to significantly reduce excessive leakage through these valves can only be performed during a major plant outage. Plant operators are aware of the leakage condition of these valves and are continuing to refine techniques based on experience to minimize the effect of this leakage when placing a Main Feedwater Pump in operation and during low feedwater flow conditions.

A similar situation concerning Main Feedwater Control Valve leakage is discussed in LER 85-020-00. However, the major contributor to that event was the belief that a Main Feedwater Control Valve isolation valve had been closed, when in fact it was partially open.

There was no damage to plant equipment or release of radioactivity as a result of this event and at no time did conditions develop that could have posed a threat to the health or safety of the public.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

July 9, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
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Arlington, Texas 76011

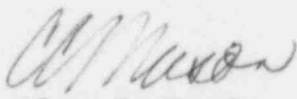
KMLNRC 85-176
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-043-00

Dear Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a) (2) (iv) concerning Engineered Safety Feature actuations.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,


for Glenn L. Koester
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a
JCurmins, w/a