



Carolina Power & Light Company
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OCT 11 1996

SERIAL: BSEP 96-0370
10 CFR 50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-324/LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-96-004

Gentlemen:

In accordance with the Code of Federal Regulations, Title 10, Part 50.73, Carolina Power & Light Company submits the enclosed Licensee Event Report. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence.

Please refer any questions regarding this submittal to Mr. Mark Turkal at (910) 457-3066.

Sincerely,

W. Levis, Director - Site Operations
Brunswick Nuclear Plant

SFT/sft

Enclosures

1. Licensee Event Report
2. Summary of Commitments

cc: Mr. S. D. Ebnetter, Regional Administrator, Region II
Mr. D. C. Trimble, NRR Project Manager - Brunswick Units 1 and 2
Mr. C. A. Patterson, Brunswick NRC Senior Resident Inspector
The Honorable H. Wells, Chairman - North Carolina Utilities Commission

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LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION
COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO
THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING
BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33),
U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE
PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET,
WASHINGTON, DC 20503.

FACILITY NAME (1)

Brunswick Steam Electric Plant, Unit 2

DOCKET NUMBER (2)

05000324

PAGE (3)

1 OF 3

TITLE (4)

Jet Pump Surveillance Not Performed Prior To Exceeding 25% Reactor Power

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	13	96	96	-- 04 --	00	10	11	96	FACILITY NAME	DOCKET NUMBER
										05000
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
			20.2201(b)		20.2203(a)(2)(v)		<input checked="" type="checkbox"/> 50.73(a)(2)(i)		50.73(a)(2)(viii)	
POWER LEVEL (10)			20.2203(a)(1)		20.2203(a)(3)(ii)		50.73(a)(2)(ii)		50.73(a)(2)(x)	
			20.2203(a)(2)(i)		20.2203(a)(3)(iii)		50.73(a)(2)(iii)		73.71	
			20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER	
			20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A	
			20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)			

LICENSEE CONTACT FOR THIS LER (12)

NAME

Steve Tabor, Sr. Analyst, Regulatory Affairs

TELEPHONE NUMBER (Include Area Code)

(910) 457-2178

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs

SUPPLEMENTAL REPORT EXPECTED (14)

YES

(If yes, complete EXPECTED SUBMISSION DATE).

☒

NO

EXPECTED
SUBMISSION
DATE (15)

MONTH

DAY

YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On September 13, 1996, during Unit 2 startup, reactor power was being maintained below the power required to place the turbine on line while making repairs to the feedwater system. As preparations were being made to restore the feedwater system, a reactor power ascension from approximately 15% to approximately 30% was commenced at 0010. This was to support generator synchronization in a timely manner once the feedwater system was returned to service. At approximately 0120 hours, with reactor power at 29%, Operations personnel recognized that the jet pump surveillance required by Technical Specification 4.4.1.2.1 had not been performed prior to exceeding 25% reactor power, which had occurred at 0101 hours. By 0127 hours, reactor power was conservatively reduced to less than 25%. The LCO ACTION statement for jet pump operability was immediately entered and executed. The required surveillance was completed by 0221 hours, and the LCO ACTION statement exited.

The failure to complete the required surveillance in accordance with the Technical Specification resulted from human performance errors. The involved individuals have received appropriate disciplinary action and a stand-down with Operations shift personnel has been completed to emphasize the significance of this event.

This event has minimal safety significance in that the required testing was recognized and performed satisfactorily within the time constraints allowed by the action statement specified in the applicable Technical Specification. This event is being reported in accordance with the requirements of 10 CFR 50.73(a)(2)(i) as a violation of Technical Specification. Technical Specification 4.0.4 requires that all surveillance requirements be performed prior to entering the applicable condition. In this case, reactor power exceeded 25% without completion of the required jet pump surveillance.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Brunswick Steam Electric Plant, Unit 2	05000324	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
		96	-- 04	-- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

TITLE

Jet Pump Surveillance Not Performed Prior To Exceeding 25% Reactor Power

INITIAL CONDITIONS

On September 13, 1996, Unit 2 reactor startup was in progress in accordance with the requirements of General Procedure (GP)-03, Unit Startup and Synchronization. Reactor power ascension in preparation for Main Turbine operation had been initiated.

EVENT NARRATIVE

During Unit 2 startup, ascension to full power was delayed to allow for feedwater heater drain system repairs. These repairs required that both trains of feedwater high pressure heaters be isolated. As the repairs were being completed and preparations were being made to return the feedwater system to service, the Senior Reactor Operator (SRO) directed that reactor power be raised from 16% to 22% to support placing the main turbine-generator on line once the feedwater system was restored to operable status. GP-03 provides the procedural guidance for raising power and placing the main turbine-generator in service.

On September 13, 1996, while increasing power to 22%, the SRO directed the reactor operator to increase reactor power to obtain two bypass valves full open as required by the main turbine operating procedure, but not to exceed 30%. This decision was based on discussions with the Shift Superintendent. At 0101 hours, reactor power exceeded 25%.

With reactor power at approximately 29%, the SRO recognized that PT-13.1, Reactor Recirculation Jet Pump Operability, required by Technical Specification 4.4.1.2.1, had not been performed prior to exceeding 25% reactor power. The applicable Limiting Condition for Operation (LCO) ACTION statement was entered due to not meeting the periodicity of the surveillance. By 0127 hours, reactor power was conservatively reduced to less than 25%. The required surveillance was completed by 0221 hours and the LCO ACTION statement exited.

This event is being reported in accordance with the requirements of 10 CFR 50.73(a)(2)(i) as a violation of Technical Specification. Technical Specification 4.0.4 requires that all surveillance requirements be performed prior to entering the applicable condition. In this case, reactor power exceeded 25% without completion of the required jet pump surveillance.

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Brunswick Steam Electric Plant, Unit 2	05000324	96	-- 04	-- 00	3 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

CAUSE

The failure to perform the required Technical Specification surveillance resulted from the following human performance problems:

Command and control of power ascension was not adequate. While implementing the startup procedure and during power ascension, the responsible SRO did not continuously review the power ascension activities and ensure that the proper activities were occurring as required.

The requirements for performing "continuous use" procedures were not followed. AP-10, Procedure Use And Adherence, requires that steps in "continuous use" procedures be read directly by the user or designated individual before step performance. In addition, Operations procedure OI-01.15, Operations Procedures, notes that "continuous use" procedures requiring sign-offs are designated by an initial blank beside the associated step. During the performance of the startup procedure, steps requiring performance of the required Technical Specification surveillance were not directly read to the individual performing the evolution nor were the steps signed off as they were completed.

The decision to increase power above 22% was not adequately assessed. Once the decision was made to supersede the established plan to only raise power to 22%, the potential liabilities of this change were not adequately assessed, nor was a follow-up pre-job briefing conducted.

CORRECTIVE ACTION

Appropriate disciplinary action was taken with the involved individuals.

A stand-down with Operations shift personnel has been completed emphasizing the significance of this event.

SAFETY ASSESSMENT

This event has minimal safety significance in that the required testing was recognized and performed satisfactorily within the time constraints allowed by the action statement specified in the applicable Technical Specification.

PREVIOUS SIMILAR EVENTS

Previous similar events involving missed Technical Specification surveillance resulting from human performance errors were reported in LERs 1-93-010 and 1-94-014.

Enclosure
List of Regulatory Commitments

The following table identifies those actions committed to by Carolina Power & Light Company in this document. Any other actions discussed in the submittal represent intended or planned actions by Carolina Power & Light Company. They are described to the NRC for the NRC's information and are not regulatory commitments. Please notify the Manager-Regulatory Affairs at the Brunswick Nuclear Plant of any questions regarding this document or any associated regulatory commitments.

Commitment	Committed date or outage
NONE	