

Commonwealth Edison Company
Byron Generating Station
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September 30, 1996



LTR: BYRON-96-0253
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Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555-001

ATTN: Document Control Desk

SUBJECT: Byron Unit 2 Cycle 7 Operating Limits Report
Byron Nuclear Power Station Unit 2
Facility Operating License NPF-66
NRC Docket Number: 50-455

Byron Unit 2 has recently completed its 6th cycle of operation and is currently preparing for Cycle 7 startup (estimated startup date is October 3, 1996). The purpose of this letter is to advise you of Commonwealth Edison's (ComEd's) review of the Cycle 7 reload under provisions of 10CFR50.59 and to transmit the Operating Limits Report (OLR) for the upcoming cycle consistent with Generic Letter 88-16.

The Byron Unit 2 Cycle 7 core, which consists of NRC approved fuel designs, was designed to operate within approved fuel design criteria, Technical Specifications and related bases such that:

- 1) core operating characteristics will be equivalent to or less limiting than those previously reviewed and accepted; or
- 2) re-analyses or re-evaluations have been performed to demonstrate that the limiting postulated UFSAR events which could be affected by the reload are within allowable limits.

Consistent with past reloads, the reload licensing analyses performed for Cycle 7 utilized NRC-approved methodologies. The fresh fuel to be loaded in Cycle 7 consists of 84 VANTAGE 5 fuel assemblies. There are 20 Region 9A assemblies with 4.80 w/o enrichment and 64 Region 9B assemblies with 4.60 w/o enrichment. The cycle-specific power distribution limits for Cycle 7 are presented in the attached OLR.

ComEd has performed a detailed review of the relevant reload licensing documents, the associated bases, and references. Based on that review, a safety evaluation was prepared, as required by 10CFR50.59, which concluded that the reload presents no unreviewed safety questions and requires no Technical Specification change. The Byron On-site Review of the reload 10CFR50.59 Safety Evaluation has been completed in accordance with station procedures.

Finally, further verification of the reload core design will be performed during startup testing. The startup tests will be consistent with Technical Specifications and testing recommended in ANS 19.6.1.

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If there are any questions regarding this submittal, please contact Marcia Lesniak, Licensing Administrator, at (630)663-6484.

Sincerely,



K. L. Koffron
Station Manager
Byron Nuclear Power Station

KLK/rp

Attachment

cc: G. F. Dick, Byron Project Manager - NRR
A. B. Beach, Regional Administrator - RIII
S. D. Burgess, Senior Resident Inspector - Byron
Office of Nuclear Facility Safety - IDNS