

# OPERATING DATA REPORT

DOCKET NO. 50-275  
 DATE 04/05/85  
 COMPLETED BY Bob Kanick  
 TELEPHONE (805)595-7351

## OPERATING STATUS

1. Unit Name: Diablo Canyon Unit 1
2. Reporting Period: March 1985
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1170
5. Design Electrical Rating (Net MWe): 1084
6. Maximum Dependable Capacity (Gross MWe): 1134\*\*\*
7. Maximum Dependable Capacity (Net MWe): 1084\*\*\*
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): N.A.
10. Reasons For Restrictions, If Any: None

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	2160	3382*
12. Number Of Hours Reactor Was Critical	650.8	1132.3	2099.5*
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	648.3	970.0	1784.0*
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1854601	2436559	3384856*
17. Gross Electrical Energy Generated (MWH)	619400	794200	1056800*
18. Net Electrical Energy Generated (MWH)	584940	722800	926805*
19. Unit Service Factor	N/A**		
20. Unit Availability Factor	N/A**		
21. Unit Capacity Factor (Using MDC Net)	N/A**		
22. Unit Capacity Factor (Using DER Net)	N/A**		
23. Unit Forced Outage Rate	N/A**		
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

100 Percent Outage - Maintenance and Startup Testing - March 29, 1985 - 1 month

25. If Shut Down At End Of Report Period, Est. Date of Startup: N/A
  26. Units In Test Status (Prior to Commercial Operation):
- |                      | Forecast      | Achieved   |
|----------------------|---------------|------------|
| INITIAL CRITICALITY  | April 1984    | April 1984 |
| INITIAL ELECTRICITY  | November 1984 | Nov. 1984  |
| COMMERCIAL OPERATION | April 1985    |            |

\* Cumulative totals started on the November 11, 1984 (Hour and date of initial electric power generation).

\*\* These sections not applicable until commencement of commercial operation.

\*\*\* These values are predictions - actual values are to be determined by operating experience during the first year of commercial operation.

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-275  
UNIT Diablo Canyon Unit I  
DATE 04/05/85  
COMPLETED BY Bob Kanick  
TELEPHONE (805)595-7351

MONTH March 1985

DAY AVERAGE DAILY POWER LEVEL  
(MWE-NET)

1	<u>783</u>
2	<u>780</u>
3	<u>780</u>
4	<u>771</u>
5	<u>780</u>
6	<u>795</u>
7	<u>644</u>
8	<u>845</u>
9	<u>965</u>
10	<u>965</u>
11	<u>895</u>
12	<u>838</u>
13	<u>994</u>
14	<u>1076</u>
15	<u>991</u>
16	<u>886</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWE-NET)

17	<u>1060</u>
18	<u>1041</u>
19	<u>1065</u>
20	<u>998</u>
21	<u>603</u>
22	<u>77</u>
23	<u>667</u>
24	<u>1116</u>
25	<u>1074</u>
26	<u>1043</u>
27	<u>1078</u>
28	<u>811</u>
29	<u>-15</u>
30	<u>-9</u>
31	<u>-10</u>

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

PAGE 1 OF 1

DOCKET NO. 50-275

UNIT NAME Diablo Canyon Unit 1DATE 04/05/85COMPLETED BY D.P. SISKTELEPHONE (805)595-7351REPORT MONTH MARCH, 1985

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutdown <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	3-21	F	21.2	G	3	85-012-00	ED	JX	Technicians trouble-shooting in the instrument racks drew an arc from the 115 volt power supply to refueling water storage tank channel 921. Arc feedback caused the reactor coolant pump breaker logic to momentarily drop out, thus triggering the reactor trip. To prevent similar feedback from causing recurrences of this, or similar events, a DCR is being initiated to provide fuse protection for the new electronic isolators recently installed in compliance with Regulatory Guide 1.97.
2	3-28	S	74.5	B	1	N/A	TB	GEN	Start-up Test 43.4, plant trip from 100 percent power.

1  
F: Forced  
S: Scheduled

2  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

3  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuation from previous month.  
5-Power reduction  
6,7,8-N/A  
9-Other

4  
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-1022)

5  
Exhibit I - Same Source

DESCRIPTION OF CHANGES TO THE RADIOACTIVE WASTE SYSTEM  
DCO-EE,EH,EC-29951

As a result of a pre-use operational efficiency and ALARA consideration evaluation of the Solid Waste Storage building, the following modifications are being made:

1. For ALARA consideration an existing rollup door and sliding shield door in the north end of the building are being removed and filled in with concrete.
2. To allow access and operations of a fork lift in the building, the openings in the south end of the building are being enlarged and an unneeded shield cell area will be removed and the exterior shield cell door opening will be filled.
3. Due to the above changes, the monorail control station will be moved.
4. For ALARA considerations, a grab sampling capability is being added to the exhaust duct.

TECHNICAL SPECIFICATION EVALUATION 6.15 MAJOR CHANGES TO  
RADIOACTIVE WASTE TREATMENT SYSTEMS

1. This modification provides a waste segregation area in the northernmost section of the building and better access to the storage area in the southern part of the building, therefore increasing the effective usage of the storage area. The change is needed to provide for the capability to store low level dry active waste as well as solid waste. Changes in building design and usage are to be reflected in the FSAR. Accidents/malfunctions associated with this modification are less severe than those described and evaluated by the FSAR. No change will occur in operation, function, or failure modes of other systems as described in the FSAR. The margin of safety as defined in the basis for Technical Specification 3/4.11.4 is not reduced since the design and operation of the modified Radwaste Storage Building will comply with the general public total dose limit.
2. See above.
3. See above, there is not involvement with other plant systems.
4. The changes do not affect the release of radioactive materials.
5. Since the changes involve the addition of more shielding, the dose in the unrestricted area will decrease. Past dose analysis indicate that this facility will have a negligible contribution to expected doses in the unrestricted areas.
6. The facility is pre-operational, therefore no actual data is available on releases. Since the changes do not involve systems-only structural modifications, no change is expected in future releases.
7. The changes involve the addition of shielding and therefor will reduce the exposure of plant personnel. Since the facility is pre-operational, an actual expected dose reduction is not available.
8. PSRC minutes of DCN approval are attached.

# PACIFIC GAS AND ELECTRIC COMPANY

PG&E +

DIABLO CANYON POWER PLANT  
P.O. Box 56 • Avila Beach, California 93424 • (805) 595-7351

R.C. THORNBERRY  
PLANT MANAGER

April 5, 1985

Office of Management Information  
and Program Control  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

RE: Docket No. 50-275  
License No. DPR-80  
Monthly Operating Report for March, 1985

Gentlemen:

Enclosed are the completed monthly operating report forms for Diablo Canyon Unit 1 for March, 1985. This report is submitted in accordance with Section 6.9.1.10 of our Technical Specifications.

Sincerely,

  
R. C. THORNBERRY

RCT:jhr

Enclosures

cc Mr. John B. Martin, Regional Administrator  
Region V - USNRC

IE24  
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MONTHLY NARRATIVE REPORT  
OF OPERATION  
AND MAJOR MAINTENANCE EXPERIENCE

This report describes the operating and major maintenance experience for the month of March, 1985. This narrative report was prepared by the plant staff and is submitted in accordance with Section 6.9.1.10 of the Plant Technical Specifications.

On March 6, 1985, PSRC approved Design Change DCO-EE,EH,EC-29951, for the Solid Radwaste Storage Building. In accordance with Technical Specification 6.15, a description of the changes is attached.

On March 13, after the 90% power plateau startup testing was completed, Unit 1 increased power to 100% for the first time.

On March 19, 1985, PSRC approved a revision to the Off-site Dose Calculation Procedure (ODCP). In accordance with Technical Specification 6.14, a copy of the changed pages (entire procedure) along with documentation of PSRC approval (Procedure History) is attached. Since the methodology for calculation and setpoint determination has not changed (Procedure History), there is no decrease in accuracy or reliability of dose calculations or setpoint determinations.

On March 21, 1985, Unit 1 experienced an automatic reactor and turbine trip.

On March 23, a net load rejection test from 100% power was performed, distinguishing Diablo Canyon as the largest Westinghouse plant ever to successfully complete this test.

The 100-hour run at 100% power was officially completed on March 28.

On March 28, 1985, NRC Region V issued a Severity Level IV Notice of Violation as part of NRC Inspection Reports 50-275/85-12 and 50-323/85-11 for Diablo Canyon Units 1 and 2. The NRC has stated that a discrepancy was observed in the compliance with Security Procedure SP 401(G).

The natural circulation/boron mixing startup test was completed on March 29 and the plant shut down, signifying the completion of the startup test program.

There was no major safety-related maintenance performed in the month of March, 1985.

No challenges to the PORVs or Steam Generator Safety Valves have been made.

No changes have been made in the Environmental Radiological Monitoring procedure.