AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE July 8, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

MONTH June, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	476.6	17	474.1
2	479.4	18	475.8
3	479.4	19	477.7
4	479.5	20	477.4
5	480.0	21	475.0
6	479.2	22	474.2
7	467.3	23	473.2
8	451.0	24	473.4
9	450.1	25	470.2
10	454.6	26	469.0
11	475.0	27	470.9
12	478.8	28	474.8
13	479.7	29	475.2
14	479.3	30	472.5
15	477.4	31	
16	474.8		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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OPERATING DATA REPORT

DOCKET NO. 50-285
DATE July 8, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: June, 1985
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	4,343.0	103,129.0
12. Number Of Hours Reactor Was Critical	720.0	4,316.6	79,596.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	720.0	4,308.7	78,976.1
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,069,152.0	6,368,248.5	100,555,015.5
17. Gross Electrical Energy Generated (MWH)	357,360.0	2,158,384.0	32,928,009.0
18. Net Electrical Energy Generated (MWH)	340.692.1	2,059,786.4	31,471,423.7
19. Unit Service Factor	100.0	99.2	76.6
20. Unit Availability Factor	100.0	99.2	76.6
21. Unit Capacity Factor (Using MDC Net)	99.0	99.2	66.3
22. Unit Capacity Factor (Using DER Net)	99.0	99.2	64.1
23. Unit Forced Outage Rate	0.0	0.0	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
1985 Refueling Shutdown is tentatively scheduled for October, 1985 with start up in December, 1985.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June, 1985

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE July 8, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns during the month of June.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending June, 1985.

1. Scheduled date for next refueling shutdown. — October, 1985

2. Scheduled date for restart following refueling. — December, 1985

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

      Technical Specifications change to accommodate increased radial peaks due to further reduction in radial leakage.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

   c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information. — September, 1985

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   Methodology Changes — June, 1985

6. The number of fuel assemblies:

a) in the core	133	assemblies
b) in the spent fuel pool	305	"
c) spent fuel pool storage capacity	729	"
d) planned spent fuel pool storage capacity	May be increased via fuel pin consolidation	"

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1996

Prepared by J R Gasper — Date July 1, 1985

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

June, 1985
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 100% power throughout June, 1985. Spent fuel assembly D005 was shipped off-site as part of a joint study by the Department of Energy and the Combustion Engineering Owners Group on high burnup fuel. The first shipment of new fuel for Cycle 10 arrived on-site June 30.

The efforts to reduce the amount of waste stored on-site continued. Shipments of waste requiring high integrity containers have reduced the inventory of long-term waste in the Auxiliary Building.

The first of the replacement low pressure feedwater heaters arrived on-site in preparation for the Fall refueling outage.

Two engineers sat for the NRC Senior Reactor Operator exam and four operators sat for NRC Reactor Operator exams during June.

A team of 13 evaluators from the Institute of Nuclear Power Operations (INPO) conducted the annual review of Fort Calhoun Station. The team interfaced with all areas of plant activities and provided a good exchange of information.

No safety valve or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-FAUD-1	Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.
SP-VA-80	Hydrogen Purge System Test. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure only checks operability of fans and cleanliness of the filters.
SP-SFS-1	Shipment of Spent Fuel in NLI 1/2 Cask. This procedure, which provided for the shipment of a spent fuel bundle, did not constitute an unreviewed safety question because technical specifications prohibiting using the crane from carrying a load over irradiated fuel were observed, an approved cask was used for the shipment and appropriate radiological requirements were addressed.

System Acceptance Committee Packages for June, 1985:

Package	Description/Analysis
EEAR FC-83-182	Dry Pipe Fire Protection in Diesel Generator Rooms. This modification converted the existing wet pipe sprinkler system to a dry pipe system. This conversion will decrease the probability of frozen piping during cold weather, thus increasing system availability. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

System Acceptance Committee Packages for June, 1985: (Continued)

Package	Description/Analysis
EEAR FC-85-53	Potable Water System Isolation Valves. This modification provided for the installation of additional valves in the potable water header. These valves will provide the capability to isolate sections of the header for maintenance. This modification has no adverse effect on the safety analysis.
EEAR FC-82-47	New Dumping of Concentrates. This modification provided for the repiping of the concentrates dump line with a more direct path to the concentrate tanks. This modification provides better dumping ability for the evaporator and original piping code standards were maintained. This modification has no adverse effect on the safety analysis.
EEAR FC-85-74	Isolation of Demineralized Water from Gas Stripper. This modification permanently isolates demineralized water feed to the gas stripper which is not used. This modification has no adverse effect on the safety analysis.
EEAR FC-84-139	TMLP Delta Noise Elimination. This modification provided for the installation of filter capacitors across the inputs to the reactor protective system thermal margin-low pressure (TMLP) calculator. These capacitors will filter out most of the disruptive noise in the signals which could lead to a spurious trip. No change to the existing operation of these circuits was made. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

System Acceptance Committee Packages for June, 1985: (Continued)

Package	Description/Analysis
EEAR FC-85-35	VHPT Dual Pot Replacement. This modification provided for the one-for-one replacement of the original RPS equipment with upgraded modules to eliminate experienced failures. The function and operation of the new modules are identical to the original equipment. This modification has no adverse effect on the safety analysis.
EEAR FC-82-108, Part 1	ATCOR Startup. This modification made only minor equipment changes that will make the waste solidification system more reliable and simpler to use. This modification has no adverse effect on the safety analysis.
EEAR FC-84-29	RPS Dual Pot Upgrade. This modification provided for the one-for-one replacement of existing system hardware with a higher reliability, less failure-prone piece of equipment. The RPS function was not altered in any way. This modification has no adverse effect on the safety analysis.
EEAR FC-84-205A	Barrier Installation. This modification did not involve safety related equipment; therefore, has no adverse effect on the safety analysis.
EEAR FC-85-51	Replace Portal Radiation Monitors. This modification replaced the existing portal radiation monitors with monitors that increased detection time, are more sensitive and easier to calibrate. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

None

F. CHANGE IN PLANT OPERATING STAFF

None

G. TRAINING

Six candidates sat for an NRC operator's license exam on June 18 (four for reactor operator and two for senior reactor operator). All six candidates passed the oral walkthrough portion of the exam. Training for Equipment Operator-Nuclear (Turbine Building) commenced. General employee, C/RP Technician, emergency plan and maintenance training continued.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

II. MAINTENANCE (Significant Safety Related)

None

L. T. Kusek

for W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

July 12, 1985
LIC-85-318

Mr. James M. Taylor, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Reference: Docket No. 50-285

Dear Mr. Taylor:

June Monthly Operating Report

Please find enclosed ten (10) copies of the June, 1985 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

R. L. Andrews
Division Manager
Nuclear Production

RLA/TPM/dao

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
American Nuclear Insurers
NRC File

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1/1