

FACILITY NAME (1)										DOCKET NUMBER (2)					PAGE (3)		
Washington Nuclear Plant - Unit 2										050003917					1 OF 8		

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)																	
0	5	0	3	8	5	8	5	5	0	2	9	0	0	0	5	3	1	8	5	0	5	0	0	0	3	9	7

OPERATING MODE (8)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § 1. Check one or more of the following: (11)									
1		20.402(b)		20.406(e)		50.73(a)(2)(iv)		73.71(b)			
POWER LEVEL (10)	0, 2, 0	20.406(a)(1)(i)		50.38(a)(1)		50.73(a)(2)(v)		73.71(c)			
		20.406(a)(1)(ii)		50.38(a)(2)		50.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.406(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(vii)(A)					
		20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(vii)(B)					
		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

NAME	TELEPHONE NUMBER	
	AREA CODE	
R.L. Koenigs, Compliance Engineer	5093	7712591
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)		

CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPRDS	
	1	1 1 1	1 1 1				1	1 1 1	1 1 1		
	1	1 1 1	1 1 1				1	1 1 1	1 1 1		

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT: Consists of 400 pages (i.e. approximately fifteen single-space typewritten lines) (16)

During normal plant shutdown for a maintenance outage at approximately 0600 hours on 5/3/85, the Rod Sequence Control System (RSCS) failed to automatically initiate to enforce a predetermined control rod programming at 20% Rated Thermal Power. Subsequently, the operating crew continued to insert control rods until 15% power was obtained. The RSCS rod inhibit mode automatically initiated at 0630 hours and the RSCS operability demonstration was not completed until approximately 0912 hours. Both actions are in violation of requirements contained in the Plant Technical Specifications.

Corrective action includes re-emphasizing Technical Specification requirements to operating crews, verifying the RSCS thermal power input and providing additional annunciation.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0500039785	02	9	00	02	OF	03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Plant Conditions

- a) Power Level 20% to 15%
- b) Plant Mode - 1

Event

During a normal plant shutdown on 5/3/85, the Rod Sequence Control System (RSCS) was inoperable for approximately 30 minutes when it did not automatically initiate at 20% power to enforce a sequence for predetermined control rod programming, but did initiate when indicated thermal power had decreased to approximately 15% at 0630 hours. Indicated Reactor power had decreased below 20% at 0600 hours and normal control rod insertion continued until 0620 hours. Technical Specifications do not allow for normal rod movement in this situation. The RSCS automatic initiation point (Low Power Setpoint) was obtained at approximately 0630 hours and the RSCS rod programming enforcement was verified at approximately 0912 hours.

During this event the Rod Worth Minimizer System was also inoperable due to an unassociated problem with the total steam flow signal.

Immediate Corrective Action

During the decrease in Reactor power from 20% to 15%, a second Licensed Reactor operator was utilized to verify control rod movement and compliance with the prescribed control rod pattern. This action complied with the requirements for inoperability of the Rod Worth Minimizer (RWM) System but not RSCS inoperability requirements. Control rod movement was terminated at 0620 hours on 5/3/85 at 15% Reactor power. No further control rod movement was initiated prior to RSCS operability being verified.

Further Corrective Action

- o Maintenance Work Requests have been issued to verify 1) Turbine first stage pressure inputs to the instruments which enable the RSCS logic, and 2) the total steam flow signal to the RWM system. These setpoints will be verified dynamically during the next plant startup.
- o The Technical Specification requirement which prohibits any control rod movement, except by SCRAM, when the RSCS is not operable and power is less than 20% of Rated Thermal Power, will be re-emphasized to all Licensed personnel via required reading of this LER prior to the next Reactor startup.
- o An annunciator, which provides the operators with visual and audible indication at the point of Low Power Setpoint initiation, will be installed prior to Reactor startup from the current maintenance outage. This will alert the Reactor operator to initiate the RSCS operability demonstration.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

- o A procedure deviation has been processed to add a cautionary note to the normal plant shutdown procedure. This note specifies that the RSCS surveillance procedure must be completed prior to lowering Reactor power below 20%.

Safety Significance

The RSCS is primarily designed to mitigate the consequences of a postulated rod drop accident from power levels of 20% and below. During the time control rods were being inserted with less than 20% Rated Thermal Power, compliance with the prescribed control rod pattern was met and verified by a second licensed operator. This precluded exceeding the designed power distribution limits and core thermal limits. Consequently, had the postulated rod drop accident occurred during this event, the resultant core thermal parameters would have been bounded by our design rod drop accident analysis.

Similar Events

Refer to LER 84-062

EIIS References

RSCS - None
RWM - None

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

May 31, 1985

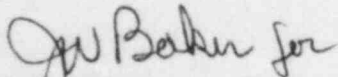
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-029

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-029 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,



C. M. Powers (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:

Licensee Event Report No. 85-029

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center, Atlanta, GA

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