

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 5					PAGE (3) 1 OF 0 2	
--	--	--	--	--	--	--	--	--	--	--------------------------------------	--	--	--	--	----------------------	--

TITLE (4) Failure to Verify Setpoint of Slack Cable Cutoff for Refueling Mast Fuel Gripper																
---	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)														
0	6	0	9	8	5	8	5	0	3	5	0	0	0	7	0	9	8	5	Brunswick Unit 2	0	5	0	0	0	3	2	4
										0 5 0 0 0																	

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																
OPERATING MODE (9) 5		20.402(b)			20.406(c)			50.73(a)(2)(iv)			73.71(b)					
POWER LEVEL (10) 0 0 0 0		20.406(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)					
		20.406(a)(1)(iii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
		20.406(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(viii)(A)								
		20.406(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)								
		20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)																
NAME M. J. Pastva, Jr., Regulatory Technician										TELEPHONE NUMBER AREA CODE 9 1 1 9 4 5 7 - 2 3 1 5						

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPDOS	

SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

T/S 4.9.6.d requires demonstration of a slack cable cutoff for the refueling mast fuel gripper when the load is less than 50 +/-25 pounds. PT-18.1 required verification of interruption of "lower" power to the main fuel grapple and receipt of the slack cable annunciator light on the refueling bridge control panel but did not verify the setpoint. This event was discovered during a technical review of PT-18.1, while troubleshooting problems with the refueling bridge main hoist slack cable cutoff function. A review of prior revisions to PT-18.1 showed this requirement had not been met in the past. Unit 1 was in a refuel/maintenance outage and Unit 2 was at power.

The subject procedural problem with PT-18.1 is attributed to inadequate technical review during development of the PT.

On 6-10-85, the PT was revised to meet T/S 4.9.6.d. This event will be reviewed by appropriate personnel to stress the importance of adequate technical review during procedure development and open lines of communications within plant organizational groups.

Appropriate Operations personnel will review this event during real-time training.

8507180306 850709
PDR ADOCK 05000325
S PDR

IE 72
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER						
Brunswick Steam Electric Plant Unit 1	0500032585	—	035	—	0	0	0	2	OF	02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On June 9, 1985, it was determined that the requirements of Technical Specification (T/S) 4.9.6.d were not met by performance of the operability test of the refueling position interlocks, Periodic Test (PT) 18.1. T/S 4.9.6.d requires a setpoint verification of the slack cable cutoff function of the refueling mast fuel gripper when the applied load is 50 +/-25 pounds. PT-18.1 provided for a functional verification of the cutoff by receipt of an interruption of "lower" power to the main fuel grapple and the slack cable alarm annunciation. This event was discovered during a technical review of PT-18.1 in response to problems with the refueling bridge main hoist on June 8, 1985. Unit 1 was in a refuel/maintenance outage and Unit 2 was at power.

A review of prior procedural revisions to PT-18.1 showed the method for verification of the slack cable cutoff remained unchanged since Revision 4 to the PT, made in January 1975. Customized T/S, in effect for Unit 2 prior to the conversion to Standardized T/S for both units in December 1977, did not specify a numerical value for verification of the cutoff. In February 1979, PT-18.1 was revised to reflect T/S reference 4.9.6; however, setpoint verification of the slack cable cutoff was not addressed in the procedure. The procedural problem involving PT-18.1 continued to exist following a plant Quality Assurance review of Revision 13.

A July 18, 1983, On-site Nuclear Safety (ONS) group review of Revisions 13 through 16 to PT-18.1 raised a concern that the PT was inadequate in meeting the requirements of T/S 4.9.6.d. A plant Operations response reflected this concern would be evaluated during an ongoing PT procedure rewrite program. During an ONS group review of proposed procedure Revision 17 to PT-18.1, on December 13, 1983, the same concern was again expressed by the ONS group. The plant Operations response to this concern reflected that the intent of T/S 4.9.6.d was met by PT-18.1. No further ONS action was taken.

On June 10, 1985, PT-18.1 was revised to meet T/S 4.9.6.d. This event will be reviewed by appropriate personnel to stress the importance of adequate technical review during procedural development and open lines of communication within and between plant organizational groups. In addition, appropriate Operations personnel will review this event during real-time training. Also, ONS will evaluate previous plant procedure audits to ascertain the probability of other similar occurrences. Based on this evaluation, appropriate corrective actions will be identified and implemented.



Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
JULY 9, 1985

FILE: B09-13510C
SERIAL: BSEP/85-1271

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-85-035

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/mcg

Enclosure

cc: Dr. J. N. Grace

IE22
11