

# OPERATING DATA REPORT

DOCKET NO. 50-364  
DATE April 4, 1985  
COMPLETED BY J. D. Woodard  
TELEPHONE (205) 899-5156

## OPERATING STATUS

1. Unit Name: Joseph M. Farley - Unit 2
2. Reporting Period: March, 1985
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 860
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 850.2
7. Maximum Dependable Capacity (Net MWe): 807.2
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
N/A

### Notes

- 1) Cumulative data since 7-30-81, date of commercial operation.

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reason: For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	2,160	32,185
12. Number Of Hours Reactor Was Critical	284.6	380.8	27,299.3
13. Reactor Reserve Shutdown Hours	0.0	0.0	138.0
14. Hours Generator On-Line	240.0	336.0	26,914.1
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	368,968	608,441	67,461,189
17. Gross Electrical Energy Generated (MWH)	114,634	192,182	22,138,464
18. Net Electrical Energy Generated (MWH)	89,028	154,478	20,973,400
19. Unit Service Factor	32.3	15.6	83.6
20. Unit Availability Factor	32.3	15.6	83.6
21. Unit Capacity Factor (Using MDC Net)	14.8	8.9	80.4
22. Unit Capacity Factor (Using DER Net)	14.4	8.6	78.6
23. Unit Forced Outage Rate	10.6	7.8	5.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

N/A

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	05-06-81	05-08-81
INITIAL ELECTRICITY	05-24-81	05-25-81
COMMERCIAL OPERATION	08-01-81	07-30-81

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DOCKET NO. 50-364UNIT 2DATE April 4, 1985COMPLETED BY J. D. WoodardTELEPHONE (205) 899-5156MONTH March, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	<u>0</u>
3	<u>0</u>	19	<u>0</u>
4	<u>0</u>	20	<u>0</u>
5	<u>0</u>	21	<u>219</u>
6	<u>0</u>	22	<u>223</u>
7	<u>0</u>	23	<u>313</u>
8	<u>0</u>	24	<u>582</u>
9	<u>0</u>	25	<u>647</u>
10	<u>0</u>	26	<u>697</u>
11	<u>0</u>	27	<u>824</u>
12	<u>0</u>	28	<u>32</u>
13	<u>0</u>	29	<u>172</u>
14	<u>0</u>	30	<u>388</u>
15	<u>0</u>	31	<u>218</u>
16	<u>0</u>		

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-364  
 UNIT NAME J.M. Farley - Unit 2  
 DATE April 4, 1985  
 COMPLETED BY J. D. Woodard  
 TELEPHONE (205) 899-5156

REPORT MONTH March, 1985

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
001	850301	S	474.6	C	1	N/A	N/A	N/A	The Cycle III-IV refueling outage continued from 1-5-85.
002	850328	F	18.9	B	3	85-008-00	N/A	N/A	The reactor tripped due to 2A steam generator low-low level caused by the loss of B main feed pump control. This was due to a printed circuit card being removed incorrectly from the 2B steam generator feed pump control cabinet. This event was caused by personnel error. The individual involved has been counseled.
003	850329	S	0.8	B	4	N/A	N/A	N/A	Turbine overspeed trip test; maintenance testing. (Reactor power at 22%)

<sup>1</sup> F: Forced  
 S: Scheduled

<sup>2</sup> Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup> Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup> Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup> Exhibit I - Same Source

# UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1985

DOCKET NO. 50-364  
 UNIT NAME J.M. Farley - Unit 2  
 DATE April 4, 1985  
 COMPLETED BY J.D. Woodard  
 TELEPHONE (205) 899-5156

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
004	850330	F	9.7	B	3	85-009-00	SG	ISV	The reactor tripped due to low-low level in 2A steam generator following the loss of steam generator feed pump 2B. During instrument calibration, an isolation valve leaked by causing the steam generator feed pump to trip due to an incorrectly indicated low vacuum condition. The isolation valve has been replaced.

<sup>1</sup>  
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 Reason:  
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 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
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 Exhibit I - Same Source

JOSEPH M. FARLEY NUCLEAR PLANT  
UNIT 2  
NARRATIVE SUMMARY OF OPERATIONS  
MARCH, 1985

The Cycle III-IV refueling outage continued into the month of March. The unit was returned to service on March 20. There were two (2) automatic shutdowns which occurred on March 28 and March 30, 1985. Following the March 30 reactor trip, atmospheric relief valves lifted for all three steam generators due to the steam dumps to the main condenser being inoperable. The steam dumps were inoperable due to a leaking isolation valve which incorrectly indicated a low vacuum condition. The generator was taken off line for performance of the turbine overspeed trip test on March 29, 1985.

The following major safety-related maintenance was performed in the month of March:

1. Performed miscellaneous corrective and preventive maintenance on diesel generators.
2. Containment tendon field anchor inspection and replacement continued. At the end of March, 79 of the 130 field anchors for the vertical tendons had been replaced. A visual inspection of a random sample of hoop and dome tendon field anchors was completed. No additional problems were found. This random sample was chosen to give a 95% confidence level that all degraded field anchors had been found.

# OPERATING DATA REPORT

DOCKET NO. 50-364  
DATE April 4, 1985  
COMPLETED BY J. D. Woodard  
TELEPHONE (205) 899-5156

## OPERATING STATUS

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2. Reporting Period: March, 1985
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### Notes

- 1) Cumulative data since 7-30-81, date of commercial operation.

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25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
  26. Units In Test Status (Prior to Commercial Operation):
- |                      | Forecast        | Achieved        |
|----------------------|-----------------|-----------------|
| INITIAL CRITICALITY  | <u>05-06-81</u> | <u>05-08-81</u> |
| INITIAL ELECTRICITY  | <u>05-24-81</u> | <u>05-25-81</u> |
| COMMERCIAL OPERATION | <u>04-01-81</u> | <u>07-30-81</u> |



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## INSTRUCTIONS

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(9/77)

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-364

UNIT NAME J.M. Farley - Unit 2

DATE April 4, 1985

COMPLETED BY L. D. Woodard

TELEPHONE (205) 899-5156

REPORT MONTH March, 1985

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
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Event Report (LER) File (NUREG-  
0161)

<sup>5</sup>  
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REPORT MONTH March, 1985

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<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

2  
Mailing Address  
Alabama Power Company  
600 North 18th Street  
Post Office Box 2641  
Birmingham, Alabama 35291  
Telephone 205 783-6090

R. P. McDonald  
Senior Vice President  
Flintridge Building



April 10, 1985

Docket No. 50-364

Director, Office of Resource Management  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

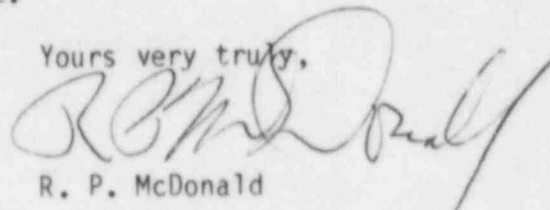
Dear Sir:

RE: Joseph M. Farley Nuclear Plant  
Unit 2  
Monthly Operating Data Report

Attached are two (2) copies of the March 1985 Monthly Operating Report for Joseph M. Farley Nuclear Plant, Unit 2, required by Section 6.9.1.10 of Appendix A of the Technical Specifications.

If you have any questions, please advise.

Yours very truly,



R. P. McDonald

RPM/KWM:sam

Enclosures

xc: Director, IE (10 copies)  
Director, RII (1 copy)

IE24  
11

JOSEPH M. FARLEY NUCLEAR PLANT  
UNIT 2  
NARRATIVE SUMMARY OF OPERATIONS  
MARCH, 1985

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