

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-213

Conn. Yankee

UNIT Haddam Neck

DATE MARCH 1985

COMPLETED BY C. B. Dean

TELEPHONE (203) 267-2556

MONTH: MARCH 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	583	17	31
2	583	18	327
3	583	19	362
4	583	20	581
5	583	21	583
6	583	22	583
7	582	23	583
8	582	24	583
9	582	25	583
10	555	26	583
11	581	27	583
12	202	28	583
13	0	29	583
14	151	30	583
15	281	31	582
16	470		

8506070248 850331  
PDR ADOCK 05000213  
R PDR

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Complete the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-213

UNIT NAME Haddam Neck

DATE March 1985

COMPLETED BY C. B. Dean

TELEPHONE (203)267-2556

REPORT MONTH MARCH 1985

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	LER RPT.	System <sup>4</sup> Code	Component <sup>5</sup> Code	Cause & Corrective Action to Prevent Recurrence
85-01	03-12-85	F	23 hrs. 33 mins.	A	3	85-007	CH	PIPEXX	High RCS pressure due to loss of main feed-water. Failure of air control line to recirculation valve.
85-02	03-16-85	F	12 hrs. 48 mins.	A	1	85-006	CH	PIPEXX	Manual trip - large leak in piping between 1B feedwater heater normal level control valve and its downstream isolation valve.

<sup>1</sup>  
F Forced  
S Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure (Explain)      H-Other(Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)

<sup>3</sup>  
Method:  
1-Manual  
2-Manual Scram  
3-Automatic Scram  
4-Other(Explain)

<sup>4</sup>  
Exhibit G-Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File  
(NUREG-0161)

<sup>5</sup>  
Exhibit 1 Same Source

REFUELING INFORMATION REQUEST

1. Name of facility

Connecticut Yankee Atomic Power Company

2. Scheduled date for next refueling shutdown.

January 4, 1986

3. Scheduled date for restart following refueling.

March 1, 1986

4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

NO

(b) If answer is yes, what, in general, will these be?

N/A

(c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Core reload design in progress.

(d) If no such review has taken place, when is it scheduled?

N/A

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

N/A

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

NO

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 157 (b) 545

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

1168

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

1994 to 1995

CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

HADDAM, CONNECTICUT

MONTHLY OPERATING REPORT NO. 85-03

FOR THE MONTH OF

MARCH 1985

## PLANT OPERATIONS

On March 10, 1985, at 0001 hours, the unit commenced a load reduction to 400 Mwe to conduct a routine turbine control valve test. The test was completed at 0230 hours and the plant returned to full power at 0530 hours.

On March 12, 1985, at 0315 hours, the switchgear room halon system discharged. This was caused by overheating of the resistor bank for the "B" condensate pump (there was no smoke or fire). At 0320 hours the unit started a load reduction to 50 percent power due to a ground in the motor stator. At 1435 hours, the plant tripped off line on high reactor coolant system pressure as load was being reduced to compensate for a broken air line on the feed pump recirculation control valve.

On March 13, 1985, at 0355 hours, the unit started a critical approach. The reactor went critical at 0611 hours, and at 1900 hours the unit was holding at 50 percent power for return to service of "B" condensate pump. On March 15, 1985, at 2255 hours, the plant commenced a load increase.

The unit reached full power on March 16, 1985, at 0410 hours. A manual plant trip was initiated at 2008 hours due to a large leak in the piping between the 1B feedwater heater normal level control valve and its downstream isolation valve.

On March 16, 1985, at 2238 hours, the unit commenced a critical approach which was suspended due to rod control malfunction. The master cyler was replaced and at 2335 hours the plant recommenced a critical approach.

The plant phased on line on March 17, 1985, at 0856 hours. After two chemistry holds at 5 percent and 25 percent, the unit commenced load increase to full power on March 18, 1985, at 0315 hours. At 0525 hours, the load increase was stopped because of a problem with low pressure steam dump relays. At 0819 hours, the plant started reducing load to work on the OPC relay. At 0910 hours, a load increase commenced to 100 percent power. Load increase was stopped at 1845 hours and the unit reduced load to 50 percent power to fix the "B" main feed pump casing gasket. On March 19, 1985, at 1316 hours, a load increase commenced and the plant reached full power at 0110 hours on March 20, 1985.

SYSTEM OR COMPONENT	INSTRUMENTATION & CONTROL MARCH 1985		EFFECT ON SAFE OPERATION	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION	SPECIAL PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY DURING REPAIR
	CAUSE	RESULT			
Power Range #4 P7 low.	Potentiometer setting off.	P-7 bi-stable low.	P7 bi-stable trips were low.	Reset potentiometer.	N/A

SYSTEM OR COMPONENT	MAINTENANCE MARCH 1985 MALFUNCTION		EFFECT ON SAFE OPERATION	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION	SPECIAL PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY DURING REPAIR
	CAUSE	RESULT			
RH-V-789A	Valve reach rod binding.	Valve hard to operate.	None	Linkage repaired.	None
FCV-1301-1	Packing leak. Wear.	Stem leakage.	None	Adjusted packing.	None
MS-TV-1211-2	Packing leak. Wear.	Stem leakage.	None	Adjusted packing.	None



CONNECTICUT YANKEE  
REACTOR COOLANT DATA  
MONTH: MARCH 1985

REACTOR COOLANT ANALYSIS	MINIMUM	AVERAGE	MAXIMUM
PH @ 25 DEGREES C	6.08E+00	6.40E+00	6.78E+00
CONDUCTIVITY (UMHOS/CM)	6.45E+00	1.13E+01	1.80E+01
CHLORIDES (PPM)	<5.00E-02	<5.00E-02	<5.00E-02
DISSOLVED OXYGEN (PPB)	<5.00E+00	<5.00E+00	<5.00E+00
BORON (PPM)	6.13E+02	6.83E+02	9.17E+02
LITHIUM (PPM)	5.73E-01	9.73E-01	1.68E+00
TOTAL GAMMA ACT. (UC/ML)	3.43E-01	6.74E-01	1.08E+00
IODINE-131 ACT. (UC/ML)	1.60E-03	6.17E-03	3.71E-02
I-131/I-133 RATIO	3.60E-01	1.76E+00	9.97E+00
CRUD (MG/LITER)	<1.00E-02	<1.00E-02	<1.00E-02
TRITIUM (UC/ML)	1.91E+00	3.49E+00	4.95E+00
HYDROGEN (CC/KG)	3.37E+01	3.54E+01	3.66E+01

AERATED LIQUID WASTE PROCESSED(GALLONS):	1.35E+05
WASTE LIQUID PROCESSED THROUGH BORON RECOVERY(GALLONS):	1.56E+05
AVERAGE PRIMARY LEAK RATE(GALLONS PER MINUTE):	2.60E-01
PRIMARY TO SECONDARY LEAK RATE(GALLONS PER MINUTE):	0.00E+00



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\* CONNECTICUT YANKEE \*  
\* HADDAM NECK PLANT \*  
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\*SINCE COMMERCIAL OPERATION 1/1/68

IF CURRENTLY SHUTDOWN, ESTIMATED STARTUP DATE..... N/A



CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

RR #1, BOX 127E, EAST HAMPTON, CONN. 06424

April 15, 1985

Docket No. 50-213

Director, Office of Management  
Information and Program Control  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Sir:

In accordance with reporting requirements, the Connecticut Yankee Haddam Neck Monthly Operating Report 85-03, covering operations for the period March 1, 1985 to March 31, 1985 is hereby forwarded.

Very truly yours,

Richard H. Graves  
Station Superintendent

RHG/sos  
Enclosures

- cc:
- (1) Director, Region I  
Division of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406
  - (2) Director, Office of Inspection and  
Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

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