

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346  
 UNIT Davis-Besse Unit 1  
 DATE May 9, 1985  
 COMPLETED BY Bilal Sarsour  
 TELEPHONE (419) 249-5000,  
Ext. 384

MONTH April 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	251
15	639
16	741

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	792
18	740
19	827
20	834
21	720
22	770
23	819
24	99
25	121
26	672
27	679
28	553
29	753
30	790
31	

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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# OPERATING DATA REPORT

DOCKET NO. 50-346  
 DATE May 9, 1985  
 COMPLETED BY Bilal Sarsour  
 TELEPHONE (419) 249-5000,  
 Ext. 384

## OPERATING STATUS

1. Unit Name: <u>Davis-Besse Unit 1</u>	Notes	
2. Reporting Period: <u>April 1985</u>		
3. Licensed Thermal Power (MWt): <u>2772</u>		
4. Nameplate Rating (Gross MWe): <u>925</u>		
5. Design Electrical Rating (Net MWe): <u>906</u>		
6. Maximum Dependable Capacity (Gross MWe): <u>904</u>		
7. Maximum Dependable Capacity (Net MWe): <u>860</u>		
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:		
<u>To obtain an acceptable offset from the Reactor Protection System (RPS)</u>		
<u>trip setpoints</u>		
9. Power Level To Which Restricted, If Any (Net MWe): _____		
10. Reasons For Restrictions, If Any: _____		
_____		
_____		

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>720</u>	<u>2,880.0</u>	<u>59,185.0</u>
12. Number Of Hours Reactor Was Critical	<u>386.7</u>	<u>1,953.8</u>	<u>34,985.3</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>4,014.1</u>
14. Hours Generator On-Line	<u>371.1</u>	<u>1,847.0</u>	<u>33,488.3</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,732.5</u>
16. Gross Thermal Energy Generated (MWH)	<u>846,236</u>	<u>4,196,587</u>	<u>79,182,009</u>
17. Gross Electrical Energy Generated (MWH)	<u>277,850</u>	<u>1,386,654</u>	<u>26,232,998</u>
18. Net Electrical Energy Generated (MWH)	<u>253,655</u>	<u>1,284,068</u>	<u>24,574,324</u>
19. Unit Service Factor	<u>51.5</u>	<u>64.1</u>	<u>56.6</u>
20. Unit Availability Factor	<u>51.5</u>	<u>64.1</u>	<u>59.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>41.0</u>	<u>51.8</u>	<u>48.3</u>
22. Unit Capacity Factor (Using DER Net)	<u>38.9</u>	<u>49.2</u>	<u>45.8</u>
23. Unit Forced Outage Rate	<u>8.1</u>	<u>1.8</u>	<u>16.6</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____		
26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April 1985DOCKET NO. 50-346UNIT NAME Davis-Besse Unit 1DATE May 9, 1985COMPLETED BY Bilal SarsourTELEPHONE (419) 249-5000, Ext. 384

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
	85 03 21	S	316.2	B	4	N/A	N/A	N/A	Maintenance outage, which began on March 21, 1985, was completed on April 14, 1985, when the unit was placed on line.
5	85 04 24	F	32.7	A	3	NP-33-85-13	JD	N/A	The Reactor Protection System (RPS) tripped the reactor on flux/delta flux/flow. See Operational Summary for further details.

<sup>1</sup> F: Forced  
S: Scheduled

<sup>2</sup> Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup> Method:  
1-Manual  
2-Manual Scram  
3-Automatic Scram  
4-Continuation from  
Previous Month  
5-Load Reduction  
9-Other (Explain)

<sup>4</sup> Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File  
(NUREG-0161)

<sup>5</sup> Exhibit I - Same Source

(9/77)

OPERATIONAL SUMMARY  
APRIL, 1985

4/1/85 - 4/13/85

The scheduled maintenance outage, which began on March 21, 1985, was completed on April 14, 1985. The outage extended longer than expected due to problems with control rod drive leaf springs being out of position.

The following are the more significant outage activities performed during the outage:

- 1) Repair of Control Rod Drives 5-3 and 2-7
- 2) Repair of main steam safety valve SP17B2
- 3) Repair generator iso phase bus phases A, B, and C
- 4) Replaced rupture disc on quench tank
- 5) Auxiliary Feedwater snubber and hanger repairs
- 6) Modification to the Steam and Feedwater Rupture Control System to alleviate Auxiliary Feedwater snubber problem

The reactor was critical at 1903 hours on April 13, 1985. The turbine-generator was synchronized on line at 0412 hours on April 14, 1985.

4/15/85 - 4/22/85

Reactor power was slowly increased and attained 89% power on April 16, 1985. Reactor power was maintained at approximately 89% until 0700 hours on April 16, 1985, when power was reduced to approximately 85% to perform turbine stop valve testing. During turbine stop valve testing, Control Rod Drive Trip Breaker C tripped. Reactor power was held at 85% power to permit continued troubleshooting of the problem.

After the completion of the turbine stop valve testing, reactor power was slowly increased to approximately 90% which was attained on April 17, 1985. Reactor power was maintained at 90% power until 1200 hours on April 17, 1985, when a manual power reduction to approximately 81% was initiated due to a problem with Condensate Pump #1. Condensate Pump #1 was taken out of service to clean its oil cooler.

Reactor power was slowly increased and attained approximately 95% power at 1600 hours on April 19, 1985. Power was maintained at this power level until 0220 hours on April 20, 1985, when manual power reduction to 93% was initiated due to a problem with Group 38 heat balance.

Reactor power was maintained at approximately 93% power until 1600 hours on April 20, 1985, when Group 38 heat balance was declared operable. Reactor power was then slowly increased to approximately 95% which was

attained at 1700 hours on April 20, 1985. Reactor power was maintained at approximately 95% power until 2400 hours on April 20, 1985 when a manual power reduction to approximately 81% was initiated due to low load requirements.

Reactor power was slowly increased and attained 95% full power on April 22, 1985.

4/23/85 - 4/25/85

Reactor power was maintained at approximately 95% power until 0353 hours on April 24, 1985, when a reactor trip occurred. The Reactor Protection System (RPS) tripped the reactor on flux/delta flux/flow. The root cause of the reactor trip was the tripping of all four flux/delta flux/flow RPS channels due to a low Reactor Coolant System (RCS) flow signal spike or an NI power spike in conjunction with an overly conservative setting of the flux/delta flux/flow setpoint.

The reactor was critical at 0613 hours on April 25, 1985. The turbine generator was synchronized on line at 1237 hours on April 25, 1985.

4/26/85 - 4/30/85

Reactor power was slowly increased to approximately 80%, which was achieved at 0500 hours on April 26, 1985. Power was not increased due to feedwater heater problems causing excessive condensate flow.

Reactor power was maintained at approximately 80% until 2300 hours on April 27, 1985, when a manual power reduction to approximately 68% was initiated due to low load requirements.

Reactor power was slowly increased to approximately 90% power, which was attained on April 29, 1985, and maintained at this power level for the remainder of the month.

REFUELING INFORMATION

DATE: April 1985

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling shutdown: Spring, 1986
3. Scheduled date for restart following refueling: Summer, 1986
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: Expect the Reload Report to require standard reload fuel design Technical Specification changes (3/4.1 Reactivity Control Systems and 3/4.2 Power Distribution Limits).

5. Scheduled date(s) for submitting proposed licensing action and supporting information: Winter, 1985
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Ans: None identified to date.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 177                      (b) 204 - Spent Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735                      Increase size by: 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1992 - assuming ability to unload the entire core into the spent fuel pool is maintained.





May 9, 1985

Log No. K85-727  
File: RR 2 (P-6-85-04)

Docket No. 50-346  
License No. NPF-3

Mr. Norman Haller, Director  
Office of Management and Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

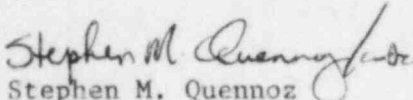
Dear Mr. Haller:

Monthly Operating Report, April 1985  
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit 1 for the month of April, 1985.

If you have any questions, please feel free to contact Bilal Sarsour at (419) 249-5000, Extension 384.

Yours truly,

  
Stephen M. Quennoz  
Plant Manager  
Davis-Besse Nuclear Power Station

SMQ/BMS/ljk

Enclosures

cc: Mr. James G. Keppler, w/1  
Regional Administrator, Region III

Mr. James M. Taylor, Director, w/2  
Office of Inspection and Enforcement

Mr. Walt Rogers, w/1  
NRC Resident Inspector

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