

NRC Form 366  
(9-83)U.S. Nuclear Regulatory Commission  
Approved OMB No. 3150-0104  
Expires: 8/31/85

## LICENSEE EVENT REPORT (LER)

Facility Name(1) Maine Yankee Atomic Power Company	Docket Number(2) 10 15 10 10 10 13 10 19	Page(3) 1 of 0 12
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Title(4) Plant Trip During Instrument Replacement
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Event Date(5) Month Day Year 0 4 3 10 8 5	LER Number(6) Sequential Number 8 5 - 0 0 3 - 0 0	Report Date(7) Month Day Year 0 5 3 0 8 5	Other Facilities Involved(8) Facility Names 0 1 5 10 10 10 1 1
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This Report is Submitted Pursuant to the Requirements of 10 CFR §  
(Check one or more of the following) (11)

Operating Mode (9) Power Level (10) 0 9 6	20.402(b)	20.405(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	73.71(b)
	20.405(a)(1)(i)	50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	Other (Specify in
	20.405(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	Abstract below
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	and in Text, NRC
	20.405(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	Form 366A)

## LICENSEE CONTACT FOR THIS LER (12)

NAME Richard S. McGrath, Nuclear Safety Engineer	Telephone Number Area Code 2 0 7 8 8 12 16 13 12 1
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## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

Cause	System	Com- ponent	Manufac- turer	Reportable to NPRDS	Cause	System	Com- ponent	Manufac- turer	Reportable to NPRDS
B	S	J	I P I S	1 8 5	Y				

## Supplemental Report Expected (14)

(If yes, complete Expected Submission Date) <input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	Expected Submission Date(15)	Month Day Year
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)  
On April 30, 1985, while operating at 96% power an automatic reactor trip on Loss of Load occurred due to main turbine trip.

The main turbine trip resulted from an Instrument and Controls technician's incorrect verification of contact positions in protective circuitry for the Turbine Driven Main Feed Pump. He mistakenly completed the pump's low suction pressure trip path with a test meter. The main turbine trips automatically on a Turbine Driven Main Feed Pump trip.

Plant safety systems responded normally following the trip.

Proper testing techniques have been reviewed and their importance stressed to Instrument and Controls personnel to prevent similar events.

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NRC Form 366A  
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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Facility Name(1)	Docket Number(2)	LER Number (6)						Page(3)	
		Year	Sequential Number		Revision Number				
Maine Yankee Atomic Power Company									
	05000309	85	003	-	00	00	2	of	02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On April 30, the plant was operating at 96% power increasing load at 2% per shift.

An Instrument and Controls Technician was working inside the Main Control Board preparing to replace the suction pressure indicator (PI) to the Turbine Driven Main Feed Pump (P) P-2C. This indicator provides a signal to a time delay relay (2) that will trip P-2C if its suction pressure drops below 260 psig for 15 seconds. The sigma Model 9262Y-21-E suction pressure indicator had failed due to a worn slide wire potentiometer. A sliding link connection (CON) to the time delay relay was opened to disable the low suction pressure trip until the indicator was replaced.

During the repair process, the technician connected a Simpson 260 test meter to the time delay relay output contacts to verify their status. He mistakenly connected one test lead to the wrong side of the open slide link. When the test meter was subsequently selected to a resistance scale a circuit around the relay output contacts was completed. This caused P-2C to trip.

The P-2C trip caused a Main Turbine (TUR) trip and reactor (RCT) trip on Loss of Load. Post trip system parameters were normal and plant safety systems responded properly.

The root cause of this event was a cognitive personnel error by the technician replacing the pressure indicator. To prevent recurrence, Instrument and Controls personnel have been instructed to use voltage measurements to determine contact positions when possible. High test meter internal resistance in the voltage mode would minimize currents through the tested circuit. It was also emphasized that technicians be constantly aware of any effects their test equipment could have on plant systems. A Plant Information Report has been written to document the event for future reference.



EDISON DRIVE  
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May 30, 1985  
MN-85-102

GDW-85-156

Director, Office of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Document Control Desk

Reference: License No. DPR-36 (Docket 50-309)

Subject: Maine Yankee Licensee Event Report 85-003-00 - Plant Trip During  
Instrument Replacement

Gentlemen:

Please find enclosed Maine Yankee Licensee Event Report #85-003-00. This report is submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,

MAINE YANKEE ATOMIC POWER COMPANY

A handwritten signature in dark ink, appearing to read 'G. D. Whittier'.

G. D. Whittier, Manager  
Nuclear Engineering and Licensing

GDW:bjp

Enclosure: two pages

cc: Mr. James R. Miller  
Dr. Thomas E. Murley  
Mr. Cornelius F. Holden

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