

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>EDWIN I. HATCH, UNIT II</b>										DOCKET NUMBER (2) <b>0 5 0 0 0 3 6 6 1</b>				PAGE (3) <b>1 OF 0 4</b>			
TITLE (4) <b>MOLDED CASE BREAKER SETPOINTS OUTSIDE OF TECH. SPECS.</b>																	
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)				
0 4	3 0	8 5	8 5	0 1 7	0 0	0 5	3 0	8 5					0 5 0 0 0				
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)															
5		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)			
POWER LEVEL (10)		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)			
0 0 0		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.406(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(vii)(A)							
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)							
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)							
LICENSEE CONTACT FOR THIS LER (12)																	
NAME										TELEPHONE NUMBER							
Steven B. Tipps, Superintendent of Regulatory Compliance										9 1 2 3 6 7 1 7 8 5 1							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																	
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS							
N/A																	
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
X YES (If yes, complete EXPECTED SUBMISSION DATE)										NO		0	8	0	1	8	5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 04/30/85 and 05/01/85, during performance of the "MOLDED CASE BREAKERS PROTECTING THE PRIMARY CONTAINMENT PENETRATION CONDUCTORS SURVEILLANCE" procedure (HNP-2-3850), plant personnel noted that Molded Case Circuit Breakers (MCCB) had trip setpoints and/or breaker locations that were contrary to the specified setpoints and/or specified locations of Tech. Specs. Table 3.8.2.6-1.

The as-found setpoints of the breakers were still sufficiently low so that the conductors would still have adequate margins of protection.

The causes were inadequate instructions for performing maintenance on the breakers (i.e., adjusting the trip setpoint when the breaker was returned to service), and implementing a design change request (DCR).

An amendment to the Tech. Specs. corrected the breaker setpoints. A request was initiated to change the Tech. Specs. specified location of the breakers for 2E51-F007 and 2E41-F002. The breaker settings were corrected in the field.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104  
EXPIRES: 8/31/85

FACILITY NAME (1)  EDWIN I. HATCH, UNIT II	DOCKET NUMBER (2)  0 5 0 0 0 3 6 6 8 5 — 0 1 7 — 0 0 0 2 OF 0 4	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 386A's) (17)

This 30 day LER is required by 10CFR50.73(a)(2)(i)(B).

On 04/30/85, with the plant in cold shutdown for a refueling outage, and during the performance of the "MOLDED CASE BREAKERS PROTECTING THE PRIMARY CONTAINMENT PENETRATION CONDUCTORS SURVEILLANCE" procedure (HNP-2-3850), plant personnel noted the following discrepancies:

1. The breaker that supplies power to a Drywell Cooling Return Fan (2T47-C002A) had an "as found" trip setpoint of approximately 75 amps. Tech. Specs. Table 3.8.2.6-1, item g.3, requires this breaker to have a trip setpoint of 20 amps. However, investigation revealed that the Tech. Specs. required setpoint was too low for the equipment being served by the breaker.
2. The breaker that supplies power to a Drywell Return Air Fan (2T47-C002B) had an "as found" trip setpoint of approximately 67.5 amps. Tech. Specs. Table 3.8.2.6-1, item g.2, requires this breaker to have a trip setpoint of 35 amps. However, investigation revealed that the Tech. Specs. required setpoint was too low for the equipment being served by the breaker.

On 05/01/85, with the plant in cold shutdown for a refueling outage, and during the performance of HNP-2-3850, plant personnel noted the following discrepancies:

1. The breaker that supplies power to a Drywell Cooling Return Air Fan (2T47-C001A) had an "as found" trip setpoint of approximately 480 amps. Tech. Specs. Table 3.8.2.6-1, item f.13, requires this breaker to have a trip setpoint of 140 amps. However, investigation revealed that the Tech. Specs. required setpoint was too low for the equipment being served by the breaker.
2. The breaker that supplies power to a Drywell Cooling Unit (2T47-C001B) had an "as found" trip setpoint of approximately 480 amps. Tech. Specs. Table 3.8.2.6-1, item f. 10, requires this breaker to have a trip setpoint of 66 amps. However, investigation revealed that the Tech. Specs. required setpoint was too low for the equipment being served by the breaker.
3. The breaker that supplies power to the RCIC Steamline Inboard Isolation MOV (2E51-F007) had an "as found" setpoint range of 90 to 270 amps. Tech. Specs. Table 3.8.2.6.-1, item e.3, requires this breaker to have a trip setpoint of 22 amps. This breaker's setpoint range wouldn't allow the required test for 22 amps.

The circuits for the RCIC Steamline Inboard Isolation MOV (2E51-F007) and the HPCI Steam Line Inboard Isolation MOV (2E41-F002) had been relocated per a Design Change Request (DCR) performed this outage as follows:

- a. the supply for 2E51-F007 was moved from Motor Control Center (MCC) 2R24-S012, compartment 19c to MCC 2R24-S012B, compartment 4A
- b. the supply for 2E41-F002 was moved from MCC 2R24-S011, compartment 14b to MCC 2R24-S011A, compartment 4A.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

However, the breaker for 2E51-F007 was not moved during the implementation of the DCR. This led to the determination that the wrong size breaker was in place for this valve.

The as found setpoints of the breakers were still sufficiently low so that the penetration conductors would still have adequate margins of protection. The circuits protected by the afore-mentioned breakers are also protected by in-line fuses. Therefore, the safety of plant equipment was not affected. Since the Unit 1 Tech. Specs. do not contain a similar section for breaker setpoints, this event is not applicable to Unit 1.

On 05/11/85, a revision was made to HNP-2-3850 that was non-conservative to the requirements of Tech. Specs. Table 3.8.2.6-1 item f.2. and f.4, (i.e., the breaker trip setpoints were changed from the Tech. Specs. specified 135 amps to 215 amps and 185 amps, respectively). 215 amps and 185 amps were the original Tech. Specs. limits. However, Tech. Specs. had been revised so that the first shutdown after 09/28/84 the setpoints would be 135 amps. On 05/13/85, during performance of HNP-2-3850, plant personnel noted that the procedural setpoints were contrary to Tech. Specs. The HNP-2-3850 procedure had been revised incorrectly, and the breakers set accordingly. The plant was not operated with the breakers set at the non-conservative setpoints.

There have been no previous similar events.

The causes of these events are as follow:

1. The wrong as found breaker trip setpoints were due to inadequate instructions for performing maintenance on the breakers:
  - a. the breakers were correctly set per the procedure and Tech. Specs.
  - b. later, when they were returned to service, the breaker would not remain closed to allow operation of the equipment it served.
  - c. a request was then generated to fix the breaker; however, adequate instructions were not provided to ensure the Tech. Specs. values were complied with.
2. The problems with 2E51-F007 and 2E41-F002 were the result of inadequate implementation of a DCR and failure of the Architect Engineer and site engineering to identify the need for a Tech. Specs. revision. When the DCR was implemented, the Document Change Request (DoCR) was not initiated at the time resulting in the circuit/Breaker location as specified in Tech. Specs. being inconsistent with its actual location following the design change.
3. The cause for the incorrect procedure revision was attributed to the unclear setpoint designation in Tech. Specs, table 3.8.2.6-1.

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Corrective actions for these events were as follow:

1. For 2T47-C002A, 2T47-C002B, 2T47-C001A, and 2T47-C001B:
  - a. An amendment to the Tech. Specs. made the following changes:
    - 1) The trip setpoint for the breaker which supplies power to 2T47-C002A was changed from 20 amps to 85 amps.
    - 2) The trip setpoint for the breaker which supplies power to 2T47-C002B was changed from 35 amps to 85 amps
    - 3) The trip setpoint for the breaker which supplies power to 2T47-C001A was changed from 140 amps to 320 amps.
    - 4) The trip setpoint for the breaker which supplies power to 2T47-C001B was changed from 66 amps to 320 amps.
  - b. The breakers were reset to correspond to the new setpoints, and returned to service.
  - c. Signs were installed on the breaker compartments (for Tech. Specs. related breakers) to notify plant personnel that the trip setpoint of the breaker so identified is Tech. Specs. related. Plant personnel were also instructed not to change the trip setpoint of these breakers.
2. For 2E51-F007 and 2E41-F002:
  - a. The original breaker for 2E51-F007 was relocated such that it corresponded to the new location of the power cables supplying this valve.
  - b. The breaker for 2E51-F007 was tested per HNP-2-3850, and returned to service.
  - c. A request was initiated to change the Tech. Specs. specified location of the breakers for 2E51-F007 and 2E41-F002.
  - d. Signs were installed on the breaker compartments (for Tech. Specs. related breakers) to notify plant personnel that the trip setpoint of the breaker so identified is Tech. Specs. related. Plant personnel were also instructed not to change the trip setpoint of these breakers.
3. A training directive was initiated to instruct the site's engineering personnel on performing thorough reviews of Design Change Request packages and to take appropriate actions if the design could affect Tech. Specs.
4. Corrective action for the review of DCRs by the Architect Engineers has not been determined. This will be included in an update report.
5. HNP-2-3850 was corrected on 05/14/85.

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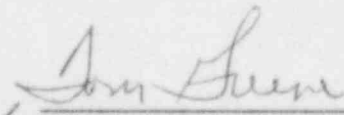
Edwin I. Hatch Nuclear Plant

May 30, 1985  
GM-85-518

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-017. This report is required by 10CFR 50.73(a)(2)(i).

  
for H. C. Nix  
General Manager

*SR1*  
HCN/STB/vlz

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