

# N.R.C. OPERATING DATA REPORT

DOCKET NO. 50-316  
 DATE 4/2/85  
 COMPLETED BY CLIMER  
 TELEPHONE 616-465-5901

## OPERATING STATUS

1. Unit Name D. C. Cook Unit 2 -----  
 2. Reporting Period Mar 1985 (notes) :  
 3. Licensed Thermal Power (MWt) 3411 :  
 4. Name Plate Rating (Gross MWe) 1133 :  
 5. Design Electrical Rating (Net MWe) 1100 :  
 6. Maximum Dependable Capacity (GROSS MWe) 1100 :  
 7. Maximum Dependable Capacity (Net MWe) 1060 -----  
 8. If Changes Occur in Capacity Ratings (Items no. 3 through 7) Since  
 Last Report Give Reasons -----

9. Power Level To Which Restricted, If Any (Net MWe) -----  
 10. Reasons For Restrictions, If Any: -----

	This Mo.	Yr. to Date	Cumm.
11. Hours in Reporting Period	744.0	2160.0	63528.0
12. No. of Hrs. Reactor Was Critical	744.0	1869.0	44949.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator on Line	744.0	1850.5	43849.1
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Therm. Energy Gen. (MWH)	2461488	6094080	139172141
17. Gross Elect. Energy Gen. (MWH)	815840	2018110	45803540
18. Net Elect. Energy Gen. (MWH)	787771	1948364	44166098
19. Unit Service Factor	100.0	85.7	71.9
20. Unit Availability Factor	100.0	85.7	71.9
21. Unit Capacity Factor (MDC Net)	99.9	85.1	68.4
22. Unit Capacity Factor (DER Net)	96.3	82.0	66.9
23. Unit Forced Outage Rate	0.0	11.7	12.6
24. Shutdowns Scheduled over Next Six Months (Type, Date, and Duration):			

25. If Shut Down At End of Report Period, Estimated Date of Startup: -----

26. Units in Test Status (Prior to Commercial Operation):  
 Forecast Achieved

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

8506070037 850331  
 PDR ADOCK 05000316  
 R PDR

IE24  
 41

AVERAGE DAILY POWER LEVEL (MWe-Net)

DOCKET NO. 50-316  
 UNIT TWO  
 DATE 4/2/85  
 COMPLETED BY CLIMER  
 TELEPHONE 616-465-5901

MONTH Mar 1985

DAY	AVERAGE DAILY POWER LEVEL	DAY	AVERAGE DAILY POWER LEVEL
1	1097	17	1094
2	1096	18	1093
3	1099	19	1080
4	1098	20	1092
5	1085	21	1094
6	1096	22	1093
7	1098	23	1087
8	1098	24	1092
9	1090	25	1091
10	1099	26	1080
11	1097	27	1090
12	1086	28	1093
13	1094	29	1093
14	1095	30	471
15	1094	31	668
16	1093		

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1985

DOCKET NO. 50-316  
 UNIT NAME D.C. Cook Unit 2  
 DATE 4-5-85  
 COMPLETED BY B.A. Svensson  
 TELEPHONE 616/465-5901

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
160	850330	F	0	A	4	N.A.	CH	VALVEX	Reactor power was reduced to 9% to permit isolation of the normal feed-water to No. 4 Steam Generator and to transfer to the auxiliary feed-water supply. This action was required due to a severe packing leak on the No. 4 Steam Generator feed-water regulating valve. The repacking was successfully completed and reactor power was returned to 100% on 850331.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I - Same Source

## UNIT SHUTDOWNS AND POWER REDUCTIONS

### INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely<sup>1</sup>. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

**NUMBER.** This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

**DATE.** This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

**TYPE.** Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

**DURATION.** Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

**REASON.** Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

**METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER.** Categorize by number designation

<sup>1</sup>Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

**LICENSEE EVENT REPORT =.** Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

**SYSTEM CODE.** The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

**COMPONENT CODE.** Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- If a component failed, use the component directly involved.
- If not a component failure, use the related component: e.g., wrong valve operated through error: list valve as component.
- If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

**CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE.** Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No.: 50-316  
Unit Name: D.C. Cook Unit 2  
Completed By: D. A. Bruck  
Telephone: (616) 465-5901  
Date: April 3, 1985  
Page: 1 of 2

MONTHLY OPERATING ACTIVITIES - MARCH, 1985

HIGHLIGHTS:

The reporting period began and ended with the Unit in Mode 1 at 100% rated thermal power. One major power reduction occurred to repair a packing leak on feedwater regulating valve, 2-FRV-240.

Total electrical generation for the month was 815,840 MWH.

SUMMARY:

- 3-05-85 At 0601, a power decrease to 97% was started for C&I Surveillance Testing. Power reached 97% at 0707.  
At 1345, a power increase to 100% was started, reaching 100% at 1430.
- 3-12-85 At 0603, a power decrease to 97% was started for C&I Surveillance Testing. Power reached 97% at 0633.  
At 1420, a power increase to 100% was started, reaching 100% at 1457.
- 3-19-85 At 0605, a power decrease at 97% was started for C&I Surveillance Testing. Power reached 97% at 0643.  
At 1615, a power increase to 100% was started, reaching 100% at 1720.
- 3-26-85 At 0607, a power decrease at 97% was started for C&I Surveillance Testing. Power reached 97% at 0700.  
At 1730 a power increase to 100% was started, reaching 100% at 1859.
- 3-30-85 At 0915, a power decrease to 9% began to repair feedwater regulating valve, 2-FRV-240.  
At 1852, a power increase to 100% began.



Docket No.: 50-316  
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Date: April 3, 1985  
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3-31-85 At 0255, the power increase was stopped a 48% due to  
quadrant power tilt calculated at 1.037.  
At 0355, because of a trailer fire inside the protected  
zone, an Unusal Event was declared.  
At 0405, the fire was out and the Unusal event  
terminated.  
At 1015, the power increase resumed, reaching 100% at  
2134.

The Control Room Cable Vault Halon System remains inoperable as  
of 1707 hours on 4-14-83. The backup CO<sub>2</sub> System remain operable.

DOCKET NO.	<u>50 - 316</u>
UNIT NAME	<u>D. C. Cook - Unit No. 2</u>
DATE	<u>4-5-85</u>
COMPLETED BY	<u>B. A. Svensson</u>
TELEPHONE	<u>(616) 465-5901</u>
PAGE	<u>1 of 1</u>

MAJOR SAFETY-RELATED MAINTENANCE

MARCH, 1985

There are no significant safety-related maintenance items to report.



**INDIANA & MICHIGAN ELECTRIC COMPANY**

Donald C. Cook Nuclear Plant  
P.O. Box 458, Bridgman, Michigan 49106

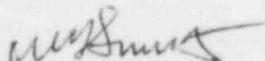
April 5, 1985

Director, Office Of Management Information  
and Program Control  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 2  
Technical Specification 6.9.1.6, the attached Monthly Operating  
Report for the Month of March, 1985 is submitted.

Sincerely,

  
W. G. Smith, Jr.  
Plant Manager

WGS:ab

Attachments

cc: J. E. Dolan  
M. P. Alexich  
R. W. Jurgensen  
NRC Region III  
B. L. Jorgensen  
R. O. Bruggee  
R. C. Callen  
S. J. Mierzwa  
R. F. Kroeger  
B. H. Bennett  
P. D. Rennix  
J. H. Hennigan  
Z. Cordero  
J. J. Markowsky  
J. F. Stietzel  
PNSRC File  
INPO Records Center  
ANI Nuclear Engineering Department

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