

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station										DOCKET NUMBER (2) 0 5 0 0 0 4 8 2 1 OF 0 2										PAGE (3) 1 OF 0 2	
TITLE (4) ESF Actuation - Feedwater Isolation, Auxiliary Feedwater Actuation																					
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)						
0 5	0 1	8 5	8 5	0 2 4	0 0	0 5	3 0	8 5							0 5 0 0 0						
OPERATING MODE (9) 3			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																		
POWER LEVEL (10) 0 1 0 0			20.402(b)			20.406(e)			<input checked="" type="checkbox"/> 80.73(a)(2)(iv)			73.71(b)									
			20.406(a)(1)(i)			80.36(a)(1)			<input type="checkbox"/> 80.73(a)(2)(v)			73.71(e)									
			20.406(a)(1)(ii)			80.36(a)(2)			<input type="checkbox"/> 80.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
			20.406(a)(1)(iii)			80.73(a)(2)(i)			<input type="checkbox"/> 80.73(a)(2)(viii)(A)												
			20.406(a)(1)(iv)			80.73(a)(2)(ii)			<input type="checkbox"/> 80.73(a)(2)(viii)(B)												
20.406(a)(1)(v)			80.73(a)(2)(iii)			<input type="checkbox"/> 80.73(a)(2)(ix)															
LICENSEE CONTACT FOR THIS LER (12)												TELEPHONE NUMBER									
NAME Merlin G. Williams - Superintendent of Regulatory, Quality, and Administrative Services												AREA CODE 3 1 6 3 6 4 1 - 8 8 3 1									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs											
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1100 CDT on May 1, 1985, an Engineered Safety Features Actuation Signal (ESFAS) was initiated when Steam Generator "A" level indication "spiked" low. This resulted in a Feedwater Isolation Signal (FWIS) and an Auxiliary Feedwater Actuation Signal (AFAS) being initiated. All required engineered safety features equipment responded properly.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of this event with the Reactor Coolant System at approximately 1900 psig and 550 degrees F.

Subsequent investigations revealed that the level indication spike was triggered by a hand-held radio transmission from the immediate area of the steam generator level transmitters.

This event posed no threat to the public health and safety.

8506070030 850530
PDR ADOCK 05000482
S PDR

IE22
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0 5 0 0 0 4 8 2 8 5 - 0 2 4 - 0 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
					0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1100 CDT on May 1, 1985, an Engineered Safety Features Actuation Signal (ESFAS) was initiated when Steam Generator "A" level indication "spiked" low, then immediately returned to normal. This resulted in a Feedwater Isolation Signal (FWIS) and an Auxiliary Feedwater Actuation Signal (AFAS) being initiated. All required engineered safety features equipment responded properly.

Investigation promptly revealed that the spike was electronic in nature, and did not represent an actual deviation in Steam Generator level. Consequently, at 1105, the systems actuated by this event were realigned to a normal configuration by plant operating procedures.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of this event with the Reactor Coolant System at approximately 1900 psig and 550 degrees F. The brief duration of this event had no appreciable affect on primary or secondary plant parameters.

Further investigation identified the cause of the level indication spike as a hand-held radio transmission from the immediate area of the Steam Generator "A" level transmitters. This characteristic has previously been experienced with the steamline pressure transmitters (Reference LER 85-022-00). As a result of these occurrences, the use of radios within the plant has been severely restricted.

This event posed no threat to the public health and safety.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

May 30, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-134
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-024-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

Glenn L. Koester
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

1522
11