

NRC FORM 366 (12-81) 10 CFR 50		U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT																																								APPROVED BY OMB 3150-0011																																					
CONTROL BLOCK																									(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)																																																						
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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10																																																																															
02 While in Mode 3, routine operator inspection revealed the absence of CVCS																																																																															
03 Letdown flow through flow orifice 2FE-0202 and a decreasing Volume Control Tank																																																																															
04 level. Since the leakage was directed so as to be collected by a system																																																																															
05 designed for this purpose, there was no impact on the health and safety of																																																																															
06 plant personnel or the public.																																																																															
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ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED NPRO-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER 25																																																																															
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CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27																																																																															
10 Leakage was caused by relief valve 2PSV-9206 sticking open following CVCS pres-																																																																															
11 sure fluctuation associated with re-routing letdown flow through 2PV-0201B. The																																																																															
12 valve was gagged closed and letdown flow re-established. After removal of the																																																																															
13 gag, the valve remained closed. An investigation into this event has determined																																																																															
14 that the lifting of Pressure Relief Valve 2PSV-9206 has been a recurring problem.																																																																															
(see attachment)																																																																															
FACILITY STATUS % POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCOVERY DESCRIPTION 32																																																																															
15 B 28 0000 29 NA B 31 Operator Inspection																																																																															
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ATTACHMENT TO LER 84-137, REVISION 2

SOUTHERN CALIFORNIA EDISON COMPANY
SAN ONOFRE NUCLEAR GENERATING STATION

UNIT NO. 2, DOCKET NO. 50-361

SUPPLEMENTAL INFORMATION FOR CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

The Letdown Control Valves (2LV0110A and 2LV0110B) have a history of cavitating in highly throttled conditions, which creates valve seat deterioration and a large amount of pressure fluctuation. In addition, the Back Pressure Valves (2PV0201A and 2PV0201B) were not capable of stabilizing the system under these conditions. This resulted in the excessive cycling of 2PSV9206 and the occasional sticking in the open position.

As corrective action, during the recent refueling outage, both the Letdown Control Valves and the Back Pressure Valves were replaced with a more reliable type of valve less susceptible to these conditions. The new valves have been fully tested and the conditions which created the excessive cycling have been stabilized.

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

May 9, 1985

U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361
Licensee Event Report No. 82-137, Revision 2
San Onofre Nuclear Generating Station, Unit 2

Reference: Letter, H. B. Ray (SCE) to R. H. Engelken (NRC), dated
November 19, 1982, "30-Day Interim Report, LER 82-137"

This submittal provides a follow-up to the referenced Licensee Event Report (LER), which was made pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-10 for San Onofre Unit 2. The occurrence involved a Limiting Condition for Operation (LCO) 3.4.5.2.d associated with identified leakage from the Reactor Coolant System (RCS).

In the referenced LER the cause and corrective actions were under investigation. This submittal provides the results of that investigation and the corrective actions which resulted.

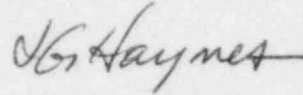
11 IE-29

Mr. J. B. Martin

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If you require any additional information, please so advise.

Sincerely,

A handwritten signature in cursive script, appearing to read "J. B. Martin", written in dark ink.

Enclosure: LER 82-137, Revision 2

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

Institute of Nuclear Power Operations (INPO)