

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Virgil C. Summer Nuclear Station										DOCKET NUMBER (2) 0 5 0 0 0 3 9 5										PAGE (3) 1 OF 0 2			
TITLE (4) Reactor Trip																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)											
0	4	2	9	8	5	0	1	3	0	0	0	5	2	9	8	5	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																					
1		20.402(b)				20.405(e)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)									
POWER LEVEL (10)		20.405(a)(1)(i)				50.38(e)(1)				50.73(a)(2)(v)				73.71(c)									
3 0		20.405(a)(1)(ii)				50.38(e)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)													
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)													
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME												TELEPHONE NUMBER											
A. R. Koon, Jr., Assoc. Mgr., Regulatory Compliance												8 0 3 3 4 5 - 5 2 0 9											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS													
X	TIG	HICUG	0810	N																			
X	SIO	IRVF	1310	N																			
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO											
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																							

On April 29, 1985 at 0130 hours, a reactor trip occurred at approximately 30% power during a plant shutdown. The reactor trip was initiated by a lo-lo steam generator water level in the "B" Steam Generator. This condition resulted from transients in deaerator tank level and main feedwater pump discharge pressure which occurred during the down power ramp. These transients caused a feedwater isolation on a low feedwater temperature (225°F) and a low feedwater flow (16%) condition. The Reactor Protection System responded as designed.

There were no adverse consequences due to this event. Due to a previous steam generator tube leak, the transient prior to the reactor trip resulted in a unmonitored release to the atmosphere from the Main Steam System. The release was conservatively calculated to be a small fraction of the allowable release limits.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED UMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Virgil C. Summer Nuclear Station	0 5 0 0 0 3 9 5	8 5	- 0 1 3	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On April 29, 1985 at 0130 hours, a reactor trip occurred at approximately 30% power during a plant shutdown. The reactor trip was initiated by a lo-lo steam generator water level in the "B" Steam Generator. This condition resulted from transients in deaerator tank level and main feedwater pump discharge pressure which occurred during the down power ramp. These transients caused a feedwater isolation on a low feedwater temperature (225°F) and a low feedwater flow (16%) condition. The Reactor Protection System responded as designed.

The feedwater transients during the down power ramp were attributed to two failures. First, the load decrease circuitry for the Main Turbine failed to function properly which resulted in a load reduction of approximately twelve (12) percent in less than three (3) minutes. This condition was further complicated by a failure of the Steam Dump System to properly respond to the transient.

There were no adverse consequences due to this event. Due to a previous steam generator tube leak, the transient prior to the reactor trip resulted in a unmonitored release to the atmosphere from the Main Steam System. The release was conservatively calculated to be a small fraction of the allowable release limits.

The Licensee has taken the following corrective actions to preclude recurrence. The load decrease circuit board for the Electro-Hydraulic Control (EHC) System of the Main Turbine was replaced. A preventive maintenance program will be established for the EHC system, which will be performed during each refueling outage. The Steam Dump System was repaired and returned to service. Additionally, a preventive maintenance program will be established for the Steam Dump System, which will be performed during each refueling outage. The Licensee's corrective action is expected to be completed prior to startup following the second refueling outage.

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

May 29, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

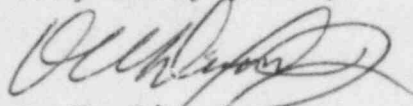
SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 85-013

Dear Sir:

Attached is Licensee Event Report #85-013 for the Virgil C. Summer Nuclear Station. This Report is submitted pursuant to the requirements of 10CFR50.73(a)(2)(iv).

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

RJB:OSB/csw
Attachment

cc: V. C. Summer
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E. H. Crews, Jr.
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