

JAN 10 1985

MEMORANDUM FOR: R. Stephen Scott, Chief
Document Management Branch
Division of Technical Information and
Document Control, ADI

FROM: J. E. Richardson, Chief
Mechanical Structural Engineering Branch
Division of Engineering Technology, RES

SUBJECT: INFORMATION COLLECTIONS IN 10 CFR 50.55a

I hereby certify that with respect to the information collection requirements in proposed 10 CFR Parts 50.55a(b)(1), (b)(2), (g)(3)(i), and (g)(3)(iii):

- the information is needed;
- the collection of information is the least burdensome necessary to comply with legal requirements and achieve program objectives;
- the collection of information is not duplicative of information otherwise accessible to the agency; and
- the collection of information has practical utility.

J. E. Richardson, Chief
Mechanical Structural Engineering Branch
Division of Engineering Technology, RES

Enclosures:

- Draft Supporting Statement
- Proposed Agency Directive

Distribution: w/o encl.
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Supporting Statement for Information Collection Requirements in
10 CFR 50.55a

1. Justification

a. Need for the Information Collection

NRC Regulations in 10 CFR 50.55a incorporate by reference Section III, Division 1, and Section XI, Division 1, of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code. These sections of the ASME Code set forth the requirements to which nuclear power plant components are designed, constructed, tested and inspected. Inherent in these requirements are certain record keeping functions.

Incorporation of the Winter 1982 Addenda, Summer 1983 Addenda, Winter 1983 Addenda, Summer 1984 Addenda, and 1983 Edition for Section III, Division 1, of the B&PV Code adds the following record keeping requirements.

Section III

- o Winter 1982 Addenda
NB-2125, Fabricated Hubbed Flanges - New provision for surface examination requires documentation of examination results.
- o Summer 1983 Addenda
No additional recordkeeping
- o Winter 1983 Addenda
NCA-3650, Design Documents for Appurtenances - Requires Design Document for each appurtenance that is to be attached to a component unless it is already included in the component Design Documents.
- o Summer 1984 Addenda
NB/NC-7240, Review of (Overpressure Protection) Report After Installation - Addendum to report required to document any modification of the installation from that used for preparation of the Overpressure Protection Report.
ND-7200, Overpressure Protection Report - Requires overpressure protection report for Class 3 components to define the protected systems and the integrated overpressure protection provided, and (ND-7240) documentation of any modification of the installation from that used for preparation of the Overpressure Protection Report.
- o 1983 Edition¹
All requirements, except those for Winter 1982 Addenda, previously incorporated in separate amendments to 10 CFR 50.55a.

¹The 1983 Edition of Section III is equivalent to the 1980 Edition, as modified by the Summer 1980 Addenda, Winter 1980 Addenda, Summer 1981 Addenda, Winter 1981 Addenda, Summer 1982 Addenda, and the Winter 1982 Addenda.

Incorporation of the Winter 1982 Addenda, Summer 1983 Addenda, and the 1983 Edition of Section XI, Division 1, of the ASME Code adds the following recordkeeping requirement.

Section XI

- o Winter 1982 Addenda
IWA-6220(b), Preparation (of Records and Reports) - Requires preparation of Owner's Report for Repairs or Replacements (Form NIS-2).
- o Summer 1983 Addenda
No additional record keeping
- o 1983 Edition²
All requirements, except those for Winter 1982 Addenda, previously incorporated in separate amendments to 10 CFR 50.55a.

The Winter 1982 Addenda of the ASME Code references ANSI/ASME NQA-1-1979, "Quality Assurance Program Requirements for Nuclear Power Plants." NQA-1-1979 is based upon the contents of ANSI/ASME N45.2-1979, "Quality Assurance Program Requirements for Nuclear Facilities" and seven daughter standards. These standards are referenced in Regulatory Guides 1.28, 1.58, 1.64, 1.74, 1.88, 1.123, 1.144, and 1.146 as providing methods acceptable for implementing certain NRC quality assurance program requirements. NQA-1-1979 incorporates no record keeping beyond that originally required by the N45 standards upon which it is based. There is, therefore, no additional record keeping burden associated with the endorsement of NQA-1-1979.

b. Practical Utility of the Information Collection

These records are used by the licensees, National Board inspectors, insurance companies, and the NRC in the review of a variety of activities, many of which affect safety. The records are generally historical in nature and provide data on which future activities can be based. NRC Inspection and Enforcement personnel can spot check the records required by the ASME Code to determine, for example, if proper inservice examination test methods were utilized.

c. Duplication With Other Collections of Information

ASME requirements are incorporated to avoid the need for writing equivalent NRC requirements. The final rule will not duplicate the information collection requirements contained in any other generic regulatory requirement.

d. Consultations Outside the NRC

No consultations.

²The 1983 Edition of Section XI is equivalent to the 1980 Edition, as modified by the Winter 1980 Addenda, Winter 1981 Addenda, and the Winter 1982 Addenda.

e. Other Supporting Information

NRC applicants and licensees have been complying with the information collection requirements of the ASME Code since 1970. No problems with these information collection requirements have been identified to the NRC by the applicants or licensees.

2. Description of the Information Collection

a. Number and Type of Respondents

In general, the information collection requirements incurred by §50.55a through endorsement of the Code apply to the owners of the 60 nuclear power plants under construction and to the owners of the 80 nuclear power plants in operation. The actual number of plants that would implement the edition and addenda addressed by the proposed revision, and thereby be affected by their information collection requirements, is dependent on a variety of factors. These factors include whether the application is for Section III or Section XI, the class and type of components involved, the dates of the construction permit and construction permit application, the schedule of the inservice inspection program, and whether the plant voluntarily elects to implement updated editions and addenda of the Code.

b. Reasonableness of the Schedule for Collecting Information

The information is generally not collected, but is retained by the licensee to be made available to the NRC in the event of an NRC inspection or audit. The preservice and inservice inspection plans are, however, submitted to the NRC for review as part of the application for an operating license.

c. Method of Collecting the Information

See Item 2(b).

d. Adequacy of the Description of the Information

The ASME Code provides listings of information required and specific forms to assist, where necessary, in documenting required information.

e. Record Retention Period

The retention period for information is in accordance with a schedule provided in the ASME Code. The retention periods for the more significant information keeping requirements specified in Item 1.a above are:

<u>Information</u>	<u>Retention Period</u> ⁽³⁾
Design document for appurtenances	Lifetime
Overpressure protection report	Lifetime
Reports for repair and replacement	Lifetime

3. Estimate of Burden

a. Estimated Hours

The information collection requirements inherent in incorporating by reference the latest edition and addenda of Section III, Division 1, and Section XI, Division 1, of the ASME Code are identified in Item 1.a above. These requirements may be categorized in terms of Section III requirements that document component/system design and the results of construction examinations, and Section XI requirements that document repairs and replacements.

The additional Section III requirements incur a one-time burden on plants under construction. The information collection requirements associated with the proposed edition and addenda are generation of the design documents for appurtenances and the overpressure protection report. Section 50.55a specifies that the Code Edition, Addenda, and optional Code Cases to be applied to reactor coolant pressure boundary, and Quality Group B and Quality Group C components must be determined by the provisions of paragraph NCA-1140 of Subsection NCA of Section III of the ASME Code. NCA-1140 specifies that the owner (or his designee) shall establish the ASME Code edition and addenda to be included in the Design Specifications, but that in no case shall the Code edition and addenda dates established in the Design Specifications be earlier than three years prior to the date that the nuclear power plant construction permit is docketed. NCA-1140 further states that later ASME Code editions and addenda may be used by mutual consent of the Owner (or his designee) and Certificate Holder. The earliest Section III addenda being addressed in the proposed rule is the Winter 1982 Addenda, since the last plant to be docketed that is still under construction was docketed in October 1974 (Palo Verde Units 1, 2, 3), there is no plant under construction for which implementation of the Section III edition and addenda specified in the proposed rule is a requirement. Plants may implement these improved rules on a voluntary basis, but unless they make that choice, there is no additional paperwork burden associated with incorporating the proposed Section III edition and addenda.

³Service lifetime of the component or system.

The additional Section XI requirements incur a burden associated with the documentation of component repairs and replacements. To facilitate this documentation, Section XI provides Form NIS-2, "Owners' Report for Repairs or Replacements." Information required by this form relates to identifying the owner and facility; identifying the components repaired or replaced and replacement components; identifying the type of work, the repair organization and by whom the work was performed; and identifying the type of tests conducted. A portion of this information, such as that to identify the owner, facility and components is already required by Form NIS-1, "Owners' Data Report for Inservice Inspections," (Form NIS-1 was part of an addenda previously incorporated by reference into Section 50.55a). Most of the remaining information required by Form NIS-2 can be obtained from the previously prepared component work/repair order. It is estimated that the time required to complete the required documentation on Form NIS-2 is one hour.

Nuclear power plants are required to update their inservice inspection programs by incorporating into their initial 120-month inspection interval requirements of the latest edition and addenda of Section XI, Division 1, that have been incorporated by reference into §50.55a as of 12 months prior to the date of issuance of the operating license; and by incorporating into successive 120-month inspection intervals requirements of the latest edition and addenda of Section XI that have been incorporated by reference as of 12 months prior to the start of a 120-month inspection interval. On this basis, most plants will at one time be required to implement the Section XI, Division 1, edition and addenda specified in the proposed rule. The number of plants that will be implementing the specified edition and addenda will grow gradually as each plant updates its inservice inspection program at the 10-year interval. Therefore, conservatively, the total number of plants that may ultimately be required to implement the specified edition and addenda is 127 (i.e., 90 operating plants and 37 plants under construction).

Inservice inspections are typically performed at the time of refueling (i.e., approximately every 18 months). The need to complete an NIS-2 form would occur as a result of a repair required by the results of an inservice inspection, or as a result of an unanticipated repair between refuelings. Typically, 2 NIS-2 forms are completed for repairs resulting from the inspection and 2 for repairs required during operation. Assuming applicability to 127 plants, and the completion of 4 NIS-2 forms by each plant every 18 months, with ten hours required to collect information and complete each form, it is estimated that the total time required by all utilities to complete the NIS-2 form is approximately 3400 hours/year.

b. Estimated Cost Required to Respond to the Collection

Based upon the hours specified in Item 3.a, it is estimated that the cost of responding to the information collection required by the Section III, Division 1, and Section XI, Division 1, edition and addenda specified in the proposed amendment to §50.55a is a total of \$170,000/year (3400 hrs x \$50/hr) for 127 plants.

c. Source of Burden Data and Method for Estimating Burden

Estimates of the number of NIS-2 forms that are completed during a year and the time required to collect the necessary information and to complete the forms, were obtained from utility staff inservice inspection specialists and NRC staff in the Office of Inspection and Enforcement (regional and headquarters) engaged in inservice inspection activities.

d. Reasonableness of Burden Estimate

The estimate of the burden is considered reasonable because of the reliable source of the burden data.

4. Estimate of Cost to the Federal Government

NRC inspection personnel who audit plant quality assurance records would include in their audit verification of the proper implementation of the NIS-2 form. The time associated with NRC inspectors verifying use of the NIS-2 form would be extremely small when the activity is performed as part of a normal quality assurance audit.

Proposed Agency Directive

NUCLEAR REGULATORY COMMISSION

10 CFR Part 50

Codes and Standards for Nuclear Power Plants

AGENCY: Nuclear Regulatory Commission.

ACTION: Proposed Rule.

SUMMARY: The Commission proposes to amend its regulations to incorporate by reference the Winter 1982 Addenda, Summer 1983 Addenda, Winter 1983 Addenda, Summer 1984 Addenda and 1983 Edition of Section III, Division 1, of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, and the Winter 1982 Addenda, Summer 1983 Addenda, and 1983 Edition of Section XI, Division 1, of the ASME Code. The sections of the ASME Code being incorporated provide rules for the construction of light-water-cooled nuclear power plant components and specify requirements for inservice inspection of those components. Adoption of these amendments would permit the use of improved methods for construction and inservice inspection of nuclear power plants.

DATES: Comment period expires _____.*

Comments received after this date will be considered if it is practical to do so, but assurance of consideration cannot be given except as to comments received on or before this date.

* A date will be inserted allowing 60 days for public comment.

ADDRESSES: Written comments or suggestions may be submitted to the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch. Copies of comments received may be examined in the Commission's Public Document Room at 1717 H Street NW., Washington, D.C.

FOR FURTHER INFORMATION CONTACT: Mr. G. C. Millman, Division of Engineering Technology, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Telephone (301)443-7860.

SUPPLEMENTARY INFORMATION: On February 7, 1983, the Nuclear Regulatory Commission published in the Federal Register (48 FR 5532) amendments to its regulation, 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," which incorporated by reference new addenda to the ASME Boiler and Pressure Vessel Code. The amendment revised § 50.55a to incorporate by reference the Winter 1981 Addenda to Division 1 rules of Section III, "Rules for the Construction of Nuclear Power Plant Components," and Division 1 rules of Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," of the ASME Boiler and Pressure Vessel Code. On November 4, 1983, the Commission published in the Federal Register (48 FR 50878) an amendment to §50.55a to incorporate by reference the Summer 1982 Addenda to Section III, Division 1, of the ASME Code. The ASME did not publish a Summer 1982 Addenda to Section XI. On March 15, 1984, the Commission published in the Federal Register (49 FR

9711) an amendment to §50.55a to, among other things, incorporate by reference those subsections of Section III which apply to the construction of Class 2 and Class 3 components.

Since publication of the last ASME Code addenda incorporated by reference in § 50.55a, the Winter 1982 Addenda, Summer 1983 Addenda, Winter 1983 Addenda, Summer 1984 Addenda, and 1983 Edition, to the ASME Code have been issued. The 1983 Edition is equivalent to the 1980 Edition, as modified by the Summer 1980 Addenda, Winter 1980 Addenda, Summer 1981 Addenda, Winter 1981 Addenda, Summer 1982 Addenda, and Winter 1982 Addenda. The Commission proposes to amend §50.55a to incorporate by reference all editions through the 1983 Edition and all addenda through the Summer 1984 Addenda that modify Section III, Division 1, and all editions through the 1983 Edition and all addenda through the Summer 1983 Addenda that modify Section XI, Division 1, of the ASME Boiler and Pressure Vessel Code. The Summer 1983 Addenda for Section XI does not include any technical requirements related to Division 1, but is being included in the reference to avoid confusion that might occur with a lack of continuity in the addenda references.

Editorial revisions are proposed to correct certain existing footnote and paragraph references that are inconsistent with the last amendment (49 FR 9711) to this rule and to simplify the language. These editorial revisions are contained entirely in §50.55a(g).

For facilities whose operating licenses were issued prior to March 1, 1976, this rule provides the effective date for implementing the inservice inspection requirements and for defining the effective edition and addenda of the Code for the start of the next one-third of a 120-month inspection interval after September 1, 1976. Since this one-third of an inspection interval has already been completed for all applicable facilities, it is proposed that the part of the rule addressing it in §50.55a(g)(4)(iii) be deleted.

Power reactors for which a notice of hearing on an application for a provisional construction permit or a construction permit had been published on or before December 31, 1970, were permitted to use the rules for construction required by power reactors that had received their construction permits prior to January 1, 1971. It is proposed that §50.55a(i) which covers this provision be deleted because it is no longer necessary. Paragraph (c)(4) provides that for these and other facilities that received a construction permit prior to May 14, 1984, the applicable Code Edition and Addenda for a component of the reactor coolant pressure boundary continue to be that Code Edition and Addenda that were required by Commission regulations for the component at the time of issuance of the construction permit.

REGULATORY ANALYSIS

The Commission has prepared a draft regulatory analysis on this proposed regulation. The analysis examines the costs and benefits of the alternatives considered by the Commission. The draft analysis is available for inspection in the NRC Public Document Room, 1717 H Street

NW, Washington, DC. Single copies of the analysis may be obtained from Mr. G. C. Millman, Division of Engineering Technology, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, D.C., 20555, Telephone (301)443-7860.

The Commission requests public comment on the draft regulatory analysis. Comments on the draft analysis may be submitted to the NRC as indicated under the ADDRESSES heading.

PAPERWORK REDUCTION ACT STATEMENT

This proposed rule amends information collection requirements that are subject to the Paperwork Reduction Act of 1980 (44 U.S.C. 3501 et seq.). This rule has been submitted to the Office of Management and Budget for review and approval of the paperwork requirements.

REGULATORY FLEXIBILITY CERTIFICATION

In accordance with the Regulatory Flexibility Act of 1980, 5 U.S.C. 605(b), the Commission hereby certifies that this rule will not, if promulgated, have a significant economic impact on a substantial number of small entities. This proposed rule affects only the licensing and operation of nuclear power plants. The companies that own these plants do not fall within the scope of the definition of "small entities" set forth in the Regulatory Flexibility Act or the Small Business Size Standards set out in regulations issued by the Small Business Administration at 13 CFR Part 121.

Since these companies are dominant in their service areas, this proposed rule does not fall within the purview of the Act.

LIST OF SUBJECTS IN 10 CFR PART 50

Antitrust, Classified information, Fire prevention, Incorporation by reference, Intergovernmental relations, Nuclear power plants and reactors, Penalty, Radiation protection, Reactor siting criteria, Reporting and recordkeeping requirements.

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974, as amended, and 5 U.S.C. 533, the NRC is proposing to adopt the following amendments to 10 CFR Part 50.

PART 50 - DOMESTIC LICENSING OF PRODUCTION AND UTILIZATION FACILITIES

1. The authority citation for Part 50 reads as follows:

AUTHORITY: Secs. 103, 104, 161, 182, 183, 186, 189, 68 Stat. 936, 937, 948, 953, 954, 955, 956, as amended, sec. 234, 83 Stat. 1244, as amended (42 U.S.C. 2133, 2134, 2201, 2232, 2233, 2236, 2239, 2282); secs. 201, 202, 206, 88 Stat. 1242, 1244, 1246, as amended (42 U.S.C. 5841, 5842, 5846), unless otherwise noted.

Section 50.7 also issued under Pub. L. 95-601, sec. 10, 92 Stat. 2951 (42 U.S.C. 5851). Sections 50.57(d) 50.58, 50.91 and 50.92 also issued

under Pub. L. 97-415, 96 Stat. 2071, 2073 (42 U.S.C. 2133, 2239). Section 50.78 also issued under sec. 122, 68 Stat. 939 (42 U.S.C. 2152). Sections 50.80-50.81 also issued under sec. 184, 68 Stat. 954, as amended (42 U.S.C. 2234). Sections 50.100-50.102 also issued under sec. 186, 68 Stat. 955 (42 U.S.C. 2236).

For the purposes of sec. 223, 68 Stat. 958, as amended (42 U.S.C. 2273), §§ 50.10(a), (b), and (c), 50.44, 50.46, 50.48, 50.54, and 50.80(a) are issued under sec. 161b, 68 Stat. 948, as amended (42 U.S.C. 2201(b)); §§ 50.10(b) and (c) and 50.54 are issued under sec. 161i, 68 Stat. 949, as amended (42 U.S.C. 2201(i)); and §§ 50.55(e), 50.59(b), 50.70, 50.71, 50.72, 50.73 and 50.78 are issued under sec. 161o, 68 Stat. 950, as amended (42 U.S.C. 2201(o)).

2. Section 50.55a is amended as follows:

Paragraph (b)(1) and the introductory text of paragraph (b)(2) are revised;

Reference to footnote 2 in paragraph (g)(1) is deleted;

References to footnote 3 in paragraphs (g)(2) and paragraphs (g)(3)(ii) and (iv) are deleted;

Paragraphs (g)(3)(i) and (iii) are revised;

Paragraph (g)(4)(iii) is deleted and reserved; and

Paragraph (i) is deleted.

§50.55a Codes and standards.

* * * * *

(b) * * *

(1) As used in this section, references to Section III of the ASME

Boiler and Pressure Vessel Code refer to Section III, Division 1, and include editions through the 1983 Edition and Addenda through the Summer 1984 Addenda.

(2) As used in this section, references to Section XI of the ASME Boiler and Pressure Vessel Code refer to Section XI, Division 1, and include editions through the 1983 Edition and Addenda through the Summer 1983 Addenda, subject to the following limitations and modifications:

* * * * *

(g) * * *

(3) * * *

(i) Components which are classified as ASME Code Class I shall be designed and be provided with access to enable the performance of inservice examination of such components and shall meet the preservice examination requirements set forth in Section XI of editions of the ASME Boiler and Pressure Vessel Code and Addenda⁶ applied to the construction of the particular component.

* * *

(iii) Pumps and valves which are classified as ASME Code Class I shall be designed and be provided with access to enable the performance of inservice testing of the pumps and valves for assessing operational readiness set forth in Section XI of editions of the ASME Boiler and

Pressure Vessel Code and Addenda⁶ applied to the construction of the particular pump or valve or the Summer 1973 Addenda, whichever is later.

(4) * * *

(iii) [Reserved]

* * * * *

Dated at _____ this _____ day of _____ 1985

For the Nuclear Regulatory Commission.

William J. Dircks
Executive Director for Operations