UPDATE REPORT — PREVIOUS REPORT SENT 1/9/84

NRC FORM 366 (12-81) 10 CFR 50 — U.S. NUCLEAR REGULATORY COMMISSION — LICENSEE EVENT REPORT — APPROVED BY OMB 3150-0011

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

(2) LICENSEE CODE: MAPPS1 — LICENSE NUMBER: 00-00000-00 (3) — LICENSE TYPE: 4 (4) — CAT: (5)

CON'T

REPORT SOURCE: L (6) — DOCKET NUMBER: 05000293 (7) — EVENT DATE: 12/13/83 (8) — REPORT DATE: 05/24/85 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

During refueling Outage VI "Appendix J Leak Rate Testing", twenty one (21) primary containment valves were found to have seat leakage in excess of that permitted by Technical Specification, Section 4.7.A.2.f. Four additional valves were removed prior to performance of the "as found leak rates".

SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
SF	E	D	VALVEX	C	A

(17) LER/RO REPORT NUMBER: EVENT YEAR 83 — SEQUENTIAL REPORT NO. 065 — OCCURRENCE CODE 03 — REPORT TYPE X — REVISION NO. 1

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
B	Z	Z	Z	0000	Y	Y	N	A395

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The cause of these excessive leakage rates were evaluated and the corrective action is specified in the attachment under remarks. All valves have been repaired and tested satisfactorily.

FACILITY STATUS (28): H — % POWER (29): 000 — OTHER STATUS (30): N/A — METHOD OF DISCOVERY (31): B — DISCOVERY DESCRIPTION (32): Appendix "J" Leak Rate Testing

ACTIVITY RELEASED (33): Z — CONTENT OF RELEASE (34): Z — AMOUNT OF ACTIVITY (35): N/A — LOCATION OF RELEASE (36): N/A

PERSONNEL EXPOSURES — NUMBER: 000 (37) — TYPE: Z (38) — DESCRIPTION (39): N/A

PERSONNEL INJURIES — NUMBER: 000 (40) — DESCRIPTION (41): N/A

LOSS OF OR DAMAGE TO FACILITY — TYPE: Z (42) — DESCRIPTION (43): N/A

PUBLICITY ISSUED (44): N — DESCRIPTION (45): N/A — NRC USE ONLY

NAME OF PREPARER: Gregory G. Belmonte — PHONE: (617) 746-7900

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PDR ADOCK 05000293
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BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

Attachment to Update LER 83-065/03X-1

VALVE NUMBER	VALVE DESCRIPTION	VALVE TYPE	FINAL TEST LEAK RATE AT 45 PSIG (SLM)	DATE PASSED	REMARKS
AO-203-1A	MSIV	20" Globe	0.1	1/16/84	See Note (1).
AO-203-2A	MSIV	20" Globe	0.1	11/17/84	See Note (1).
AO-203-1B	MSIV	20" Globe	0.1	11/16/84	See Note (1).
AO-203-2B	MSIV	20" Globe	0.1	11/16/84	See Note (1).
AO-203-1C	MSIV	20" Globe	0.1	11/17/84	See Note (1).
AO-203-1D	MSIV	20" Globe	0.1	11/16/84	See Note (1).
AO-203-2D	MSIV	20" Globe	0.1	11/16/84	See Note (1).
58A	Feedwater Check Inboard	18" Check	0.1	9/2/84	Replaced Soft Seat & Hinge Pin
58B	Feedwater Check Inboard	18" Check	0.1	9/7/84	Replaced Soft Seat & Hinge Pin
62A	Feedwater Check Outboard	18" Check	0.1	9/5/84	Replaced Soft Seat & Hinge Pin
62B	Feedwater Check Outboard	18" Check	0.1	9/14/84	Replaced Soft Seat & Hinge Pin
MO-1201-80	Rx Water Cleanup Return	4" Globe	0.1	11/23/84	Replaced with 4" Stainless Steel Valve from Anchor Darling
AO-7011-B	Drywell Equipment Drain	2" Plug	0.1	11/9/84	Replaced Valve & Actuator from Atwood & Morrill
31-203	Instrument Air	3" Check	0.7	10/31/84	Replaced with 3" Stainless Steel Valve from Anchor Darling

VALVE NUMBER	VALVE DESCRIPTION	VALVE TYPE	FINAL TEST LEAK RATE AT 45 PSIG (SLM)	DATE PASSED	REMARKS
AO-4356	Instrument Air	3" Gate	3.0	10/31/84	Replaced with 3" Stainless Steel Valve from Anchor Darling
AO-5033B	Drywell Purge Inlet	4" Butterfly	6.2	11/10/84	Repacked Valve
MO-1001-29B	RHR Injection Inboard	18" Gate	0.1	11/29/83	Ground Seat and Replaced Wedge
MO-1001-26A	Containment Spray	10" Gate	0.2	10/13/84	Replaced Torque Switch and Lubricated Stem
AO-220-44	Reactor Water Sample	1" Globe	5.2	11/25/84	Replaced Diaphragm & Straightened Bent Stem
AO-5035A	Drywell Purge	8" Butterfly	6.2	11/10/84	Replaced with 8" AO Butterfly Valve from Clow Corp.
AO-5035B	Drywell Purge	8" Butterfly	6.2	11/10/84	Replaced with 8" AO Butterfly Valve from Clow Corp.
2301-45	HPCI Turbine Exh. Check	20" Check	0.9	9/26/84	Replaced Hinge Pin, Hinge, End-Cap Gasket, and Bonnet Gasket. See Note (4).
CV-5065-13	H_2/O_2 Analyzer	1" Globe	N/A	N/A	See Note (2).
CV-5065-20	H_2/O_2 Analyzer	1" Globe	N/A	N/A	See Note (2).
CV-5065-14	H_2/O_2 Analyzer	1" Globe	N/A	N/A	See Note (2).
CV-5065-21	H_2/O_2 Analyzer	1" Globe	N/A	N/A	See Note (2).

NOTES:

(1) Rework on the Main Steam Isolation Valves included new poppets with extended nose pieces, machining of new guide area below the seats, new stems, installation of anti-rotation devices on stem, rewelding body guides, seats, and polishing of seats.

(2) Four valves were removed prior to performance of "As-Found Leak Rates." Previous test reveals no evidence that they would have had any significant contribution to total leak rate. Valves replaced with 1" Valcor Corp. valves.

(3) All reading of 0.1 represents the minimum detectable reading of the instruments.

(4) Not an Appendix J related valve. Inadvertently reported as an Appendix "J" valve failure in the original LER.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
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WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

May 24, 1985
BECo Ltr. #85-094

Dr. Thomas E. Murley
Regional Administrator, Region I
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Docket Number 50-293
License DPR-35

Gentlemen:

The attached update Licensee Event Report 83-065/03X-1, "Appendix 'J' Related Valves," is hereby submitted in accordance with the previous requirements of Pilgrim Nuclear Power Station Technical Specification Section 6.9.B.2.b.

This followup report identifies each valve for which maintenance was initiated as a result of Appendix J Leak Rate Testing during Refuel Outage VI.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,

W D Harrington

W. D. Harrington

GB:caw

Enclosure: LER 83-065/03X-1

cc: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Standard BECo LER Distribution

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