UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SACRAMENTO MUNICIPAL UTILITY DISTRICT

DOCKET NO. 50-312

RANCHO SECO NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66
License No. DPR-54

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Sacramento Municipal Utility District (the licensee) dated September 9, 1982, as revised December 28, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-54 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 66 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 28, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 66

FACILITY OPERATING LICENSE NO. DPR-54

DOCKET NO. 50-312

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove	Insert
vi	vi
ix	ix
4-54	4-54
4-55	4-55
--	4-55a
--	4-57a
--	4-57b

LIST OF TABLES

4.17.4 Acceptance Criteria

3. Degraded Tube means a tube containing imperfections ≥20% of the nominal wall thickness caused by degradation.

4. Defective Tube means a tube containing an imperfection ≥40% of the nominal tube wall thickness unless higher limits are shown acceptable by analysis. Defective tubes shall be plugged.

5. Tube Inspection means an inspection of the steam generator tube from the point of entry completely to the point of exit (except as noted in 4.17.2c).

b. The steam generator shall be determined OPERABLE after completing the corresponding actions required by Table 4.17-2.

4.17.5 Reports

a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.

b. The results of the steam generator tube inservice inspection shall be included in the Annual Operating Report for the period in which this inspection was completed. This report shall include:

1. Number and extent of tubes inspected.

2. Location and percent of wall-thickness penetration for each indication of an imperfection.

3. Identification of tubes plugged.

c. Results of steam generator tube inspections which fall into Category C-3 and require notification of the Commission shall be reported pursuant to Specification 6.9 prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

4.17.6 OTSG Auxiliary Feedwater Header Surveillance

On the first refueling outage following the 1983 refueling outage, and at the 10-year ISI, the following inspections will take place:

Surveillance Standards

4.17.6 a. Visual inspections of the secured Internal Header, attachment welds and external headers thermal sleeves will be made through selected openings, and will be performed in such a manner that the known cracks in the Internal Header will be inspected.

b. Selected special interest peripheral tubes, designated in Table 4.17-3 , will be Eddy Current inspected but shall not be considered a part of the Eddy Current inspection that is conducted pursuant to Technical Specification 4.17.1 through 4.17.5.

4.17.7 Inspection Acceptance Criteria and Corrective Actions

a. Video taped inspections of the known cracks performed during the initial discovery will be compared with the crack configuration found during this surveillance. This comparison will allow a determination to be made as to whether or not the crack has propagated.

b. If any inspected special interest peripheral tube indicates clearance (less than 1/4") or greater than 40% through wall indications, it will be plugged.

4.17.8 Reports

A report of these inspections will be prepared and incorporated in the subsequent Monthly Report to the NRC.

Bases

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The surveillance requirements of steam generator tubes are based on a modification of B&W - Standard Technical Specifications dated June 1, 1976. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

Wastage-type defects are unlikely with AVT chemistry treatment of the secondary coolant. However, even if a defect should develop in service, it will be found during scheduled inservice steam generator tube examinations. Plugging will be required for defective tubes. Steam generator tube inspections of operating plants have demonstrated the capability to reliably detect degradation that has penetrated 20% of the original tube wall thickness.

Bases (Continued)

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, these results will be reported to the Commission pursuant to Specification 6.9 prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection and revision of the Technical Specifications, if necessary.

The visual and eddy current inspections provide the capability of determining the success of the internal aux feedwater header stabilization by monitoring the long term effects to tube integrity and crack propagation. The inspections will focus on known crack growth, new crack identification (if any), and tube effects in localized areas near the internal header brackets. Additionally, visual inspection of the external header thermal sleeves will provide assurance that the new design header will not introduce additional problems by demonstrating sleeve integrity.

TABLE 4.17-3

OTSG Auxiliary Feedwater Header Surveillance

OTSG A Special Interest Tubes

Row	Tube No
5	1, 46
6	2, 3, 49, 50, 51
7	1, 2, 53, 54
8	1, 2, 56, 57
44	1, 119
45	1, 120
46	1, 121
47	1, 122
48	1, 123
49	1, 124
103	1, 124
105	1
106	1, 119
107	1, 120
108	1, 119
144	1, 2, 56, 57
145	1, 2, 53, 54
146	2, 3, 49, 50, 51
147	1, 46

TABLE 4.17-3 (Continued)

OTSG Auxiliary Feedwater Header Surveillance

OTSG B Special Interest Tubes

Row	Tube No
5	1, 46
6	2, 3, 49, 50
7	1, 2, 53, 54
8	1, 2, 56, 57
44	1, 119
45	1, 120
46	1, 121
47	122
48	1, 123
49	1, 124
103	1, 124
104	123
105	122
106	1, 119
107	1, 120
108	1, 119
144	1, 2, 56, 57
145	1, 2, 53, 54
146	2, 3, 49, 50, 51
147	1, 46