

May 24, 1985

MEMORANDUM FOR: Richard W. Starostecki, Director  
Division of Reactor Projects  
Region I

FROM: Hugh L. Thompson, Jr., Director  
Division of Licensing

SUBJECT: CONTAINMENT ISOLATION VALVES UNDER  
ADMINISTRATIVE CONTROL

Your assistance is requested in conducting a review of the enclosed submittal from Maine Yankee Atomic Power Company dated April 23, 1985. The product expected as a result of your review is a completed safety evaluation. Your reviewers should use the Standard Review Plan (SRP) and Standard Technical Specifications (STS) as guidance in determining acceptance criteria, recognizing, of course, that, for operating reactors, the criteria in these documents are not requirements.

In accordance with NRR Office Letter No. 44, each safety evaluation performed by a technical division shall have a separate SALP input provided. For purposes of these reviews, the Regional personnel involved are considered part of the technical divisions. Therefore we are requesting that your forwarding memorandum contain a SALP input for each safety evaluation performed.

Work for this review effort should be charged under TAC 57635. The requested completion date is August 31, 1985. Please notify me as soon as possible if this completion date is acceptable. Any contact with the licensee on this review effort or any additional information deemed necessary should be obtained through the NRR project manager. The project manager for this plant is P. Sears and he can be reached at 492-7458 (FTS).

Original Signed by  
Hugh L. Thompson, Jr.

Hugh L. Thompson, Jr., Director  
Division of Licensing

Enclosure:  
As stated

cc w/enclosure  
J. Thoma  
D. Haverkamp  
K. Heitner  
P. Sears

DISTRIBUTION:

DOCKET FILE  
ORB#3 RDG  
NRC PDR  
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KHeitner  
PSears  
HLThompson  
GCLainas

CONTACT:  
P. Sears

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D:DL  
HThompson  
5/23/85

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P PDR

May 24, 1985

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Programs, Region I

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HThompson  
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ATOMIC POWER COMPANY •

April 23, 1985  
MN-85-76

EDISON DRIVE  
AUGUSTA, MAINE 04336  
(207) 623-3521

Proposed Change #114

Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Document Control Desk

References: (a) License No. DPR-36 (Docket No. 50-309)  
(b) MYAPCo Letter USNRC dated March 15, 1985 (MN-85-56)  
Schedule for Submittal of Post Accident Sampling  
Technical Specifications  
(c) USNRC Letter to MYAPCo dated April 11, 1985  
Post Accident Sampling Technical Specifications for  
Procedures

Subject: Containment Isolation Valves Under Administrative Control

Gentlemen:

In Reference (b) above, Maine Yankee agreed to submit proposed changes to our Technical Specifications addressing sampling and analysis of primary coolant and containment atmosphere under post accident conditions. Following discussions with your Staff and receipt of Reference (c), Maine Yankee has concluded that it is preferable to address post accident sampling in our Emergency Operating Procedures rather than in Technical Specifications.

However, in order to adequately test portions of the post accident sampling system, it is necessary to propose changes to our Technical Specifications permitting realignment of certain containment isolation valves.

Therefore, pursuant to the Commission's Rule and Regulations (10 CFR 10.90), Maine Yankee requests that Appendix A of our Operating License be amended as follows:

Remove existing page 3.11-2 and replace with new page 3.11-2.

This change to Section 3.11 adds to the list of manual containment isolation valves which may be repositioned under administrative control without taking specific compensatory measures. This will permit flow testing of post accident hydrogen purge and analysis system, as well as obtaining containment atmosphere samples for analysis.

Each of these new valves is in a small bore line and would be opened only briefly for the purpose of demonstrating flow or obtaining a sample. Hence, the probability of an accident occurring during the time the valves are open is small.

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United States Nuclear Regulatory Commission  
Attention: Document Control Desk

Page Two  
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We have, therefore, concluded that this proposed change would not significantly increase the consequences of a previously analyzed accident nor would it increase the probability of an accident occurring. Neither does this change create the possibility of a previously unanalyzed accident. Hence no significant hazards considerations are involved.

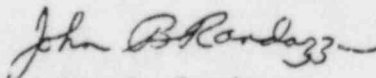
This change has been reviewed and approved by the Plant Operation Review Committee. As well, the Nuclear Safety and Audit Review Committee has reviewed this proposed change.

A representative of the State of Maine is being informed of this change request by a copy of this letter.

Enclosed is our check for \$150.00 to cover the application fee for this proposed change.

Very truly yours,

MAINE YANKEE ATOMIC POWER COMPANY



John. B. Randazza  
Executive Vice President

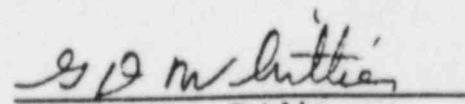
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Enclosure:

cc: Mr. James R. Miller  
Dr. Thomas E. Murley  
Mr. Cornelius F. Holden  
Mr. Clough Tappan

STATE OF MAINE           )  
                                  )ss  
COUNTY OF LINCOLN    )

Then personally appeared before me, John B. Randazza, who being duly sworn did state that he is Executive Vice President of Maine Yankee Atomic Power Company, that he is duly authorized to execute and file the foregoing request in the name and on behalf of Maine Yankee Atomic Power Company, and that the statements therein are true to the best of his knowledge and belief.



Notary Public  
G.D. VANTIER  
NOTARY PUBLIC, MAINE  
MY COMMISSION EXPIRES OCTOBER 19, 1991

Exception:

1. Manual containment isolation valves may be repositioned under the administrative controls provided prior compensatory measures are taken to isolate the penetration. Compensatory measures which may be taken include closing other valves or installing blind flanges to isolate a penetration. Compensatory measures which are in effect for longer than seventy-two hours must meet the same design criteria as the original containment isolation valve.
2. The following manual containment isolation valves may be repositioned under administrative control without compensatory measures to isolate the penetration:

RH-4, 6, 7, 26  
 MS-48, 50, 68, 70, 88, 90, 239, 241, 243  
 PAP-1, 4, 23, 24, 25  
 HPD-4, 5, 9, 10, 14, 15  
 IA-212, 213, 218, 219, 220

Remedial Action:

With one or more automatic or remotely operated containment isolation valves inoperable, maintain at least one automatic or remotely operated isolation valve operable in each affected penetration that is open, and within 4 hours either:

1. Restore the inoperable valve to operable status, or
  2. Isolate the affected penetration by use of at least one manual, remotely operated or deactivated automatic isolation valve secured (tagged) in the closed position or by use of a blind flange. Compensatory measures which are in effect for longer than seventy-two hours must meet the same design criteria as the original containment integrity valve.
- C. The reactor shall not be critical if the containment internal pressure exceeds 3 psig.
- D. On-line containment purge.

On-line purging of containment is not allowed under the conditions of Specification 3.11.B above, unless the following conditions are satisfied:

1. Prior to the depressurization of the containment for on-line purge, the two purge supply, two exhaust valves and one exhaust bypass valve will be cycled to test the automatic closure feature using both the Containment Gaseous Activity and Primary Vent Stack Gaseous Activity closure signals. The operation of the inner and outer supply and exhaust

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