

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 1 OF 2																					
TITLE (4) Pressure Boundary Leakage																															
EVENT DATE (6)						LER NUMBER (8)				REPORT DATE (7)						OTHER FACILITIES INVOLVED (9)															
MONTH			DAY			YEAR			YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH			DAY			YEAR			FACILITY NAMES				DOCKET NUMBER(S)			
																									0 5 0 0 0 0 0 0 0 0						
0 2			0 3			8 5			8 5		0 1 1		0 1		0 6 2 7			8 5							0 5 0 0 0 0 0 0 0 0						
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																													
0 0 2																															
POWER LEVEL (10)		20.402(b)																													
0 1 0 3		20.408(a)(1)(i)																													
		20.408(a)(1)(ii)																													
		20.408(a)(1)(iii)																													
		20.408(a)(1)(iv)																													
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		20.408(a)																													
		50.38(a)(1)																													
		50.38(a)(2)																													
		50.73(a)(2)(i)																													
		50.73(a)(2)(ii)																													
		50.73(a)(2)(iii)																													
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		50.73(a)(2)(vi)																													
		50.73(a)(2)(vii)(A)																													
		50.73(a)(2)(vii)(B)																													
		50.73(a)(2)(viii)																													
		50.73(a)(2)(ix)																													
		73.71(b)																													
		73.71(a)																													
		X OTHER (Specify in Abstract below and in Text, NRC Form 356A)																													
		50.72(b)(2)(i)(A)																													
LICENSEE CONTACT FOR THIS LER (12)																															
NAME										TELEPHONE NUMBER																					
R. L. Koenigs, Compliance Engineer										510 9 31 7 71-1 2 510 1 1																					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2279																															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																					
X	D	J P I S E	I	I	N																										
X	D	O P I S E	I	I	N																										
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)																					
YES (if yes, complete EXPECTED SUBMISSION DATE)										NO																					

ABSTRACT (LIMIT TO 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 2/3/85, during a routine inspection, two 3/4" lines were found to have leakage. These 3/4" drain lines formed part of the primary system pressure boundary. The reactor was shutdown and the lines were repaired per ASME requirements and returned to service.

During the May/June 1985 Maintenance Outage, additional supports were added to these lines. Ten additional drain line configurations judged most likely to have a similar problem were examined and no additional indications of crack propagation were found. A detailed study has concluded that the observed failures were due to low load high cycle fatigue and no generic remedial design modifications are anticipated.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7 8 5	0 1	1 1	0 1	0 1 2 OF 2	

TEXT (If more space is required, use additional NRC Form 305A's) (17)

Plant Conditions

- a) Power Level 3%
b) Reactor Mode 2

Event

On 2/3/85, during a routine inspection required by Plant startup procedures, two 3/4" lines were observed with slight leakage. The leakage, in both cases, was far below the Plant limit of 5 gpm (unidentified) and had not caused a detectable reading on plant leakage monitors, which would have required action per the Technical Specifications. Both lines [RHR(55)45-2 and RFW(55)-4-6] are 3/4" drain lines with two unsupported isolation valves and are considered part of the primary system pressure boundary.

Immediate Corrective Action

The Reactor was shutdown and both lines were repaired per ASME Section XI procedures. Subsequent non-destructive tests verified the repair adequacy and the lines were returned to service.

Future Corrective Action

- o During the May/June 1985 Maintenance Outage, additional supports for these lines were designed and installed. The supports will enhance the resistance of these drain lines to operational fatigue loads.
- o Ten additional drain line configurations judged most likely to have a similar problem were examined during the current maintenance outage. The lines were examined by dye penetrant means and in no case were any linear indications found.
- o A detailed failure analysis report was prepared and the failure mechanism was concluded to be low load high cycle fatigue. The report also concludes that during normal operation, vent/drain line stress levels are so low that a theoretically infinite fatigue life is expected. Generic remedial design modifications are not considered necessary at this time.

Safety Significance

The allowable leakage limits for the reactor coolant system have been based on predicted and experimentally observed pipe crack behavior. The probability is small that cracks associated with the leakage observed would have propagated rapidly. However, even had these lines completely failed, the Plant Leakage Detection System would have alerted Plant operators to shutdown the Reactor. At no time would this have impaired the Plant's ability to maintain the Reactor in a safe condition. There was no hazard to the health and safety of the public or plant personnel.

Similar Events

Not Applicable

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

June 27, 1985

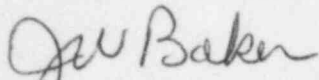
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-011-01

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-011-01 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, action taken to preclude recurrence and provides supplemental information to LER-85-011.

Very truly yours,



C.M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:1a

Enclosure:
Licensee Event Report No. 85-011-01

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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