

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 8 7										PAGE (3) 1 OF 0 1	
TITLE (4) Four (4) Fire Dampers Not Included in Surveillance Procedures.																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)						
									SSES - Unit 2						0 5 0 0 0 3 8 8						
0 4	0 4	8 5	8 5	0 1	2	0 0	0 5	2 4	8 5						0 5 0 0 0						
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																			
5		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)							
POWER LEVEL (10)		20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)							
0 0 0		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
		20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)											
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME L.A. Kuczynski - Nuclear Plant Specialist, Level III										TELEPHONE NUMBER											
										AREA CODE 7 1 7 5 4 2 - 1 3 7 5 9											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC											
A	*	B D M P R	4 1 1	N																	
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)				MONTH		DAY		YEAR			
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 4, 1985, it was determined by the Fire Protection System Engineer and one of the Station's Resident NRC Inspectors that four (4) fire dampers in the Standby Gas Treatment System (SGTS) ductwork were not included in the appropriate 18-month surveillance procedures. These dampers should have been included in the surveillance because they close on high temperature (actuator: fusible links) to isolate SGTS ductwork to prevent spread of fire from one fire zone to another via the ductwork as it passes through the Reactor Building and Control Structure. The dampers were added to the procedures. The inspection consists of a visual examination of each damper and its associated hardware to ensure that it is open, unobstructed and free from visible damage. Although the proper notification of the need to establish an hourly fire watch in the affected areas was given to Operations personnel and the appropriate Limiting Conditions for Operation were noted, it could not be verified that the watches were actually implemented on April 4, 1985. The Fire Watch Log (controlled by Administrative Procedure AD-QA-143) for that date does not reflect the plant areas affected. Subsequently, one of the dampers was inspected and a fire watch was established and will remain in effect until the remaining dampers are inspected. The methods/processes used to assure identification and surveillance of safety-related dampers will be reviewed and revised as necessary to mitigate recurring incidents.

\* Fire Protection.

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PDR ADOCK 05000387  
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# Pennsylvania Power & Light Company

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May 24, 1985

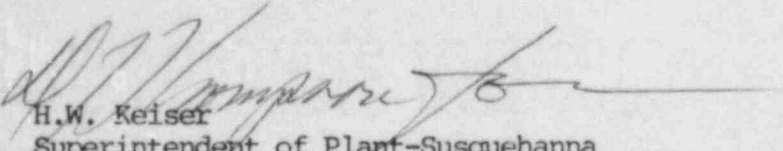
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SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 85-012-00  
ER 100450 FILE 841-23  
PLAS- 080

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Docket No. 50-387  
License No. NPF-14

Attached is Licensee Event Report 85-012-00. This event was determined reportable per 10CFR50.73(a)(2)(i), in that four (4) safety-related fire dampers were not included in the procedures used to comply with Technical Specification Surveillance Requirement 4.7.7.1. The oversight was corrected.



H.W. Keiser  
Superintendent of Plant-Susquehanna

LAK/pjg

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