

PORTLAND GENERAL ELECTRIC COMPANY
EUGENE WATER & ELECTRIC BOARD
AND
PACIFIC POWER & LIGHT COMPANY

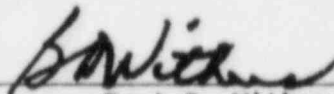
TROJAN NUCLEAR PLANT

Operating License NPF-1
Docket 50-344
License Change Application 99, Rev. 2

This License Change Application updates the heatup and cooldown curves (Figures 3.4-2 and 3.4-3) of the Technical Specifications, Appendix A to Operating License NPF-1 for the Trojan Nuclear Plant, in compliance with Appendices G and H of 10 CFR 50.

PORTLAND GENERAL ELECTRIC COMPANY

By



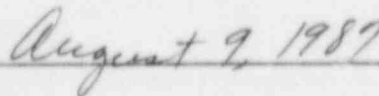
Bart D. Withers
Vice President
Nuclear

Subscribed and sworn to before me this 12th day of July 1985.



Carol A. Hodgdon
Notary Public of Oregon

My Commission Expires:



LICENSE CHANGE APPLICATION 99, REVISION 2

The following changes to Appendix A of Facility Operating License NPF-1 are requested (proposed replacement pages are provided as Attachment 1).

Pages 3/4 4-25 and 3/4 4-26. Changes to Section IV.A.2 of 10 CFR 50, Appendix G, require that when the core is not critical, and pressure exceeds 20 percent of the preservice system hydrostatic test pressure (620 psi), that the temperature of the closure flange regions must exceed the reference temperature of the material in those regions by at least 120°F for normal operation. The heatup and cooldown rate pressure-temperature limit curves of Figures 3.4-2 and 3.4-3 have been revised to reflect this requirement. For the purpose of the change, the reference temperature for the head flange was determined to be 20°F from Technical Specification Table B 3/4.4-1.

Page 3/4 4-27. The capsule withdrawal schedule in 10 CFR 50, Appendix H was changed to require compliance with ASTM E 185-82 for capsules withdrawn after July 26, 1983. The withdrawal schedule in ASTM E 185-82 is based on effective full power years (EFPY) rather than calendar years as is required by the current Technical Specification. WCAP-9469, "Analysis of Capsule U From Portland General Electric Company Trojan Reactor Vessel Radiation Surveillance Program", provided a recommended removal schedule for the remaining capsules based on ASTM E 185-79 requirements. The withdrawal schedule requirements of ASTM E 185-82 did not change from the 1979 version of this code, and therefore the withdrawal schedule provided is in compliance with ASTM E 185-82.

Page B 3/4 4-5. The heatup and cooldown curves shown in Figures 3.4-2 and 3.4-3 are applicable for the first 15 EFPY rather than 12 EFPY as stated in Paragraphs 3 and 4 on this page. A paragraph was added to discuss the bases for the change to Figures 3.4-2 and 3.4-3.

Page B 3/4 4-9. In accordance with the change to 10 CFR 50, Appendix H, ASTM E 185-82 is now the accepted version of this standard. The reference to ASTM E 185-73 as the version for removing and evaluating the capsule specimens is therefore being changed to ASTM E 185-82. Additionally, in Paragraph 3, Table 4.4-3 is incorrectly referenced and is being changed to read Table 4.4-5.

REASON FOR CHANGE

On Friday, May 27, 1983, the NRC published in the Federal Register a Final Rule amending the fracture toughness requirements for light water nuclear power reactors and the requirements for reactor vessel material surveillance programs. These rule changes affected 10 CFR 50 Sections 50.12, 50.55 (a), 50.60 and Appendices G and H. As a result of the change to 10 CFR 50, Appendix H, Technical Specification Table 4.4-5 must be revised to reflect the schedule changes for specimen capsule removal.

The Bases for Technical Specification 4.4.9.1 are being revised in accordance with the changes made to Appendices G and H and to correct typographical errors. Finally, Figures 3.4-2 and 3.4-3 and the Technical Specification Bases for these Figures are being revised to reflect new 10 CFR 50, Appendix G, pressure-temperature limits.

SAFETY/ENVIRONMENTAL EVALUATION

Table 4.4-5 in Technical Specification 4.4.9.1 will be changed to reflect the new capsule withdrawal schedule required by 10 CFR 50, Appendix H. This schedule corresponds with the schedule recommended for the Trojan Nuclear Plant in WCAP-9469, which was prepared in accordance with ASTM E 185-79. The schedule requirements in ASTM E 185-82, which is the referenced version of the standard in 10 CFR 50, Appendix H, are the same as those in ASTM E 185-79, and therefore, the proposed schedule is in compliance with 10 CFR 50, Appendix H. This change is administrative in nature, and does not result in an unreviewed safety question. The changes to the Technical Specification Bases are administrative in nature, and therefore, do not decrease the margin of safety as defined therein.

Figures 3.4-2 and 3.4-3 are being revised to comply with the new requirements in 10 CFR 50, Appendix G. For the temperature range of approximately 100°F to 150°F the pressure limitation will be lower than previously required. This change does not increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously analyzed in the FSAR nor does it create the possibility of an accident or malfunction of equipment of a different type than any previously evaluated. The change does not reduce the margin of safety as defined in the Bases of the Technical Specifications.

SIGNIFICANT HAZARDS DETERMINATION

This license change results in very minor changes to facility operations and has been initiated in order to comply with NRC regulations. The change is administrative in nature and does not involve a significant hazards consideration. In fact, the schedule for irradiated specimen withdrawal has been moved up such that any potentially damaging radiation effects to the reactor vessel will be discovered at an earlier date than was previously scheduled, thus increasing the margin of safety.

BASIS FOR DETERMINATION OF AMENDMENT CLASS

It was determined that this change, as originally submitted on January 27, 1984, resulted in a Class II Amendment as it was then defined in 10 CFR 170.22. The Class II Amendment fee of \$1,200 was submitted at that time.