

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 2 0 6				PAGE (3) 1 OF 0 2							
TITLE (4) SHUTDOWN FROM POWER OPERATED RELIEF VALVE BLOCK VALVE CV-530 INOPERABILITY																					
EVENT DATE (5)				LER NUMBER (6)				REPORT DATE (7)				OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YFAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)								
0 6	0 5	8 5	8 5	0 0 8	0 0	0 6	2 8	8 5					0 5 0 0 0								
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																			
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)							
POWER LEVEL (10)		0 9 2				20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)			
		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
		20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)											
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME J. G. HAYNES, STATION MANAGER										TELEPHONE NUMBER 7 1 4 4 9 2 - 7 7 0 0											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC											
X	A/B	I/S/V	A 3 9 1	Y																	
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO									

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 4, 1985, at 2235, with Unit 1 at 92% power, the block valve (CV-530) to Power Operated Relief Valve (PORV) CV-546 failed to indicate closed during surveillance testing. This valve had been maintained closed due to CV-546 leakage. CV-530 stroke tests confirmed the valve was not closing and the valve was declared inoperable and Technical Specification 3.5.1, Action Statement "B" was entered. The block valve (CV-531) to PORV CV-545 tested satisfactorily and remained operable.

A Containment entry at 0130, on June 5, 1985, confirmed CV-530 was partially open. As a result, at 0135 shutdown was initiated in accordance with Technical Specification 3.5.1, Action Statement "B". The unit entered Hot Standby at 0532.

Failure of CV-530 was due to a leaking actuator diaphragm. The diaphragm was replaced, tested, and CV-530 was returned to service at 1255 on June 5, 1985. Therefore, CV-530 was out of service for 14 hours and 20 minutes.

There are no reasonable or credible circumstances under which this event could have been more severe.

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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQ. NUMBER	REV. NUMBER		
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 1	0 5 0 0 0 2 0 6	8 5	- 0 0 8	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On June 4, 1985, at 2235, with Unit 1 at 92% power, block valve CV-530 (EIIS Component Code ISV) to Power Operated Relief Valve (EIIS Component Code RV) (PORV) CV-546 failed to indicate closed during surveillance testing. Normal operation requires this block valve to remain open, but it had been maintained closed due to CV-546 leakage. Control operators stroked the valve several times to obtain a fully closed indication, but were unsuccessful. Due to the lack of positive indication, the block valve was declared inoperable, and Technical Specification 3.1.5, Action Statement "B" was entered. The other block valve (CV-531) to PORV CV-545 was tested satisfactorily and remained operable.

At 0130, on June 5, 1985, an authorized containment entry was made to verify actual condition of the valve position indicators and CV-530. It was confirmed that the valve position indicators were functioning properly and CV-530 was partially open. As a result, at 0135, shutdown was initiated in accordance with Technical Specification 3.1.5, Action Statement "B". At 0532, the unit entered Mode 3.

CV-530 failed due to a leaking actuator diaphragm Model 70-13-1D as manufactured by MUESCO Engineering. The leak originated from the bolt circle area of the diaphragm where a portion of the diaphragm was found missing. This area of the diaphragm acts as the actuator housing gasket. Sealant, which is not normally required, had been found on both sides of the housing. Sealant had been installed to prevent leakage from the backup nitrogen system air supply. On the area of the missing portion of the diaphragm, sealant had filled in the gap, which indicates that a portion of the diaphragm was already missing when it was installed on September 16, 1984. It was in this area where the air was found leaking. Our investigation could not determine the cause of the missing diaphragm portion. However, it should be noted that CV-530 tested satisfactorily after installation of the diaphragm on September 16, 1984, and on succeeding scheduled operations and maintenance surveillances conducted prior to this event.

The diaphragm was replaced, tested, and CV-530 was returned to service at 1255 on June 5, 1985. Therefore, CV-530 was out of service for 14 hours and 20 minutes. As further corrective actions, the actuator diaphragms of CV-530 and CV-531 will be replaced every refueling outage, but the actuator overhaul will remain at the current frequency of every sixth refueling. In addition, the actuator diaphragm on CV-531 will be inspected and replaced at the next available outage.

During the event, Reactor Coolant System (RCS) leakage past PORV CV-546 was monitored, and there was no indication of increased leakage and RCS integrity was maintained. Neither the health and safety of plant personnel nor the health and safety of the public was affected by this event.

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

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SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

June 28, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Subject: Docket No. 50-206
30-Day Report
Licensee Event Report No. 85-008
San Onofre Nuclear Generating Station, Unit 1

Pursuant to 10 CFR 50.73(a)(2)(i)(A), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the pressurizer Power Operated Relief Valve (PORV). Neither the health and safety of plant personnel nor the health and safety of the public was affected by this event.

If you require any additional information, please so advise.

Sincerely,

J.G. Haynes/sem

Enclosure: LER No. 85-008

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
A. J. D'Angelo (USNRC Resident Inspector, Unit 1)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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