

50-416



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

October 22, 1996

Mr. Joseph J. Hagan  
Vice President, Operations GGNS  
Entergy Operations, Inc.  
P. O. Box 756  
Port Gibson, MS 39150

SUBJECT: EVALUATION OF THE FIRST 10-YEAR INTERVAL INSPECTION PROGRAM PLAN,  
REVISION 1 TO RELIEF REQUEST I-00014 FOR GRAND GULF NUCLEAR STATION,  
UNIT 1 (TAC NO. M96293)

Dear Mr. Hagan:

By letters dated August 12 and September 26, 1996, you requested relief for the Grand Gulf Nuclear Station, Unit 1 (GGNS) from the requirements of Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) for the first 10-year inservice inspection (ISI) interval in accordance with Paragraphs 50.55a(g)(5)(iv) and (g)(6)(i) of 10 CFR Part 50. You requested that Relief Request No. I-00014 be revised to incorporate ASME Code Case N-498-1 as alternative testing in the GGNS pressure testing program, such that the reactor pressure vessel (RPV) will be subjected to pressure testing in accordance with the requirements of ASME Code Section XI or ASME Code Case N-498-1.

The revision to the relief request also proposed increasing manual ultrasonic examination of the RPV to nozzle welds for enhanced volumetric coverage of the weld and adjacent base metal in accessible areas to supplement the examinations previously conducted under the provisions of the original relief request. The Relief Request No. I-00014 was previously approved in the staff's letter of December 30, 1987.

The ISI for ASME Code Class 1, 2, and 3 components at GGNS shall be performed in accordance with Section XI of the ASME Code and applicable addenda as required by 10 CFR 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55a(g)(6)(i). For 10 CFR 50.55a(a)(3), it states that alternatives to the requirements of paragraph (g) may be used, when authorized by the Commission, if (i) the proposed alternatives would provide an acceptable level of quality and safety or (ii) compliance with the specified requirements would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.

Pursuant to 10 CFR 50.55a(g)(4), ASME Code Class 1, 2, and 3 components (including supports) shall meet the requirements, except the design and access provisions and the pre-service examination requirements, set forth in the ASME Code, Section XI to the extent practical within the limitations of design,

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geometry, and materials of construction of the components. The regulations require that inservice examination of components and system pressure tests conducted during the first 10-year interval and subsequent intervals comply with the requirements in the latest edition and addenda of Section XI of the ASME Code incorporated by reference in 10 CFR 50.55a(b) 12 months prior to the start of the 10-year interval, subject to the limitations and modifications listed therein. The applicable edition of Section XI of the ASME Code for the GGNS first 10-year ISI interval is the 1977 edition through the summer 1979 addenda.

Pursuant to 10 CFR 50.55a(g)(5), if the licensee determines that conformance with an examination requirement of Section XI of the ASME Code is not practical for its facility, information shall be submitted to the Commission in support of that determination and a request made for relief from the ASME Code requirement. After evaluation of the determination, pursuant to 10 CFR 50.55a(g)(6)(i), the Commission may grant relief and may impose alternative requirements that are determined to be authorized by law, will not endanger life, property, or the common defense and security, and are otherwise in the public interest, giving due consideration to the burden upon the licensee that could result if the requirements were imposed.

As stated in the enclosed Safety Evaluation, the staff concludes that the revision to Relief Request No. I-00014 to include the system leakage test in ASME Code Case N-498-1 for GGNS is an administrative change and you have the option to perform either the 10-year hydrostatic test or the system leakage test of the RPV as allowed by the code case. The alternative of the use of the code case provides an acceptable level of quality and safety. Also, the manual examination of more nozzle-to-vessel welds in the revision of the relief request provides an increased level of quality and safety in comparison to that of the automated examination. Therefore, pursuant to 10 CFR 50.55a(a)(3)(i), relief described in Revision 1 to Relief Request No. I-00014 is granted by the Commission.

Sincerely,



William D. Beckner, Project Director  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: Safety Evaluation

cc w/encl: See next page

geometry, and materials of construction of the components. The regulations require that inservice examination of components and system pressure tests conducted during the first 10-year interval and subsequent intervals comply with the requirements in the latest edition and addenda of Section XI of the ASME Code incorporated by reference in 10 CFR 50.55a(b) 12 months prior to the start of the 10-year interval, subject to the limitations and modifications listed therein. The applicable edition of Section XI of the ASME Code for the GGNS first 10-year ISI interval is the 1977 edition through the summer 1979 addenda.

Pursuant to 10 CFR 50.55a(g)(5), if the licensee determines that conformance with an examination requirement of Section XI of the ASME Code is not practical for its facility, information shall be submitted to the Commission in support of that determination and a request made for relief from the ASME Code requirement. After evaluation of the determination, pursuant to 10 CFR 50.55a(g)(6)(i), the Commission may grant relief and may impose alternative requirements that are determined to be authorized by law, will not endanger life, property, or the common defense and security, and are otherwise in the public interest, giving due consideration to the burden upon the licensee that could result if the requirements were imposed.

As stated in the enclosed Safety Evaluation, the staff concludes that the revision to Relief Request No. I-00014 to include the system leakage test in ASME Code Case N-498-1 for GGNS is an administrative change and you have the option to perform either the 10-year hydrostatic test or the system leakage test of the RPV as allowed by the code case. The alternative of the use of the code case provides an acceptable level of quality and safety. Also, the manual examination of more nozzle-to-vessel welds in the revision of the relief request provides an increased level of quality and safety in comparison to that of the automated examination. Therefore, pursuant to 10 CFR 50.55a(a)(3)(i), relief described in Revision 1 to Relief Request No. I-00014 is granted by the Commission.

Sincerely,

Original signed by

William D. Beckner, Project Director  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: Safety Evaluation

cc w/encl: See next page

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Grand Gulf Nuclear Station

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